

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Fort Calhoun Station, Unit No. 1										DOCKET NUMBER (2) 0 5 0 0 0 2 8 5				PAGE 13 1 OF 0 2									
TITLE (4) Load Over the RCS																							
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)													
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES N				DOCKET NUMBER(S) 0 5 0 0 0										
0	7	0	2	8	4	8	4	0	1	5	0	0	0	8	0	1	8	4	0	5	0	0	0
OPERATING MODE (9) 5		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following) (11)																					
POWER LEVEL (10) 0 0 0		20.402(b)				20.406(a)				80.73(a)(2)(iv)				73.71(b)									
		20.406(a)(1)(i)				80.36(a)(1)				80.73(a)(2)(v)				73.71(c)									
		20.406(a)(1)(ii)				X 80.36(a)(2)				80.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)									
		20.406(a)(1)(iii)				80.73(a)(2)(i)				80.73(a)(2)(vii)(A)													
		20.406(a)(1)(iv)				80.73(a)(2)(ii)				80.73(a)(2)(vii)(B)													
		20.406(a)(1)(v)				80.73(a)(2)(iii)				80.73(a)(2)(ix)													
LICENSEE CONTACT FOR THIS LER (12)																							
NAME Lawrence T. Kusek, Supervisor-Operations Fort Calhoun Station, Unit No. 1										TELEPHONE NUMBER AREA CODE 4 0 2 4 2 6 - 4 0 1 1													
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																							
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRPDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRPDS													
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR							
YES (If yes, complete EXPECTED SUBMISSION DATE)												X NO											

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

A load of approximately 250 pounds was carried by the polar crane over the reactor coolant system when the fluid in the pressurizer was greater than 225 degrees Fahrenheit violating Technical Specification 2.11(1). Pressurizer temperature and pressure at the time of the incident were approximately 388°F and 220 psia, respectively. When the violation was discovered, the load was suspended above grating over a steam generator cavity. The load was set on a concrete support and the polar crane was placed in its parked position. To prevent similar events from occurring in the future, a procedure change has been made to RCS vent and leak test operating instructions to tag out the polar crane prior to exceeding 225 degrees Fahrenheit in the pressurizer.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Fort Calhoun Station, Unit No. 1	0 5 0 0 0 2 8 5	8 4	— 0 1 5	— 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On July 2, 1984 at 1400, the polar crane carried a skip box weighing approximately 250 pounds over part of the reactor coolant system when the fluid in the pressurizer was greater than 225 degrees Fahrenheit violating Technical Specification 2.11(1). When the violation was discovered, the load was suspended above grating over a steam generator cavity. The load was set on a concrete support and the polar crane was placed in its parked position. To prevent similar events from occurring in the future, a procedure change has been made to RCS vent and leak test operating instructions to tag out the polar crane prior to exceeding 225 degrees Fahrenheit in the pressurizer.

Loads are not to be allowed over the pressurized reactor coolant system to preclude dropping objects which could rupture the boundary of the reactor coolant system allowing loss of coolant and over-heating of the core. At the time of the incident the pressurizer was at approximately 388 degrees and 220 psia. The RCS was at approximately 185 degrees and being heated up with one reactor coolant pump in operation. Shutdown cooling was in operation with low pressure and high pressure safety injection pumps available. The load was 12 feet above the grating over the steam generator cavity. Had the load fallen, it is unlikely it would have penetrated the grating. Had this occurred and the RCS boundary been ruptured, the available safety injection system components would have been able to mitigate possible over-heating of the core.

Omaha Public Power District
1623 Harney Omaha, Nebraska 68102
402/536-4000

August 1, 1984
FC-397-84
LIC-84-252

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

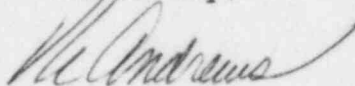
Reference: Docket No. 50-285

Gentlemen:

Licensee Event Report for
the Fort Calhoun Station

Please find attached Licensee Event Report 84-015 dated August 1, 1984. This report is being submitted per requirements of 10 CFR 50.73.

Sincerely,



R. L. Andrews
Division Manager
Nuclear Production

RLA/jmm

Attachment

cc: Mr. Richard P. Denise, Director
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& Engineering Programs
U. S. Nuclear Regulatory Commission
Region IV
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Arlington, Texas 76011

INPO Records Center
Mr. E. G. Tourigny, Project Manager

SARC Chairman
PRC Chairman
Mr. L. A. Yandell, Senior Resident
Inspector
Fort Calhoun File (2)

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