



WASHINGTON PUBLIC POWER SUPPLY SYSTEM

P.O. Box 968 • 3000 George Washington Way • Richland, Washington 99352-0968 • (509) 372-5000

September 20, 1995
GO2-95-188

Docket No. 50-397

Mr. L. J. Callan
NRC Regional Administrator
U.S. NRC, Region IV
611 Ryan Plaza Drive, Suite 400
Arlington, Texas 76011-8064

Dear Mr. Callan:

Subject: **WNP-2, OPERATING LICENSE NPF-21
FACILITY OPERATION AT REACTOR CORE POWER LEVELS
IN EXCESS OF FULL POWER**

This is a notification to the NRC Regional Administrator in accordance with license condition 2.F.

On September 19, 1995, preliminary results of an independent measurement of the reactor feedwater flow rate indicated that the instruments in use provide a feedwater flow rate which is 2.46% less than actual. The feedwater flow test was requested by the Supply System and conducted by General Electric in accordance with SIL 452 recommendations. Thus, with reactor power indicating 100% of rated, power may have been as high as 102.4%. The time period is still under investigation.

The feedwater flow test will be repeated to verify the test results. The second test is expected to start on Thursday, September 21. The preliminary results from the second test are expected by September 27 and the final results from both test are expected to be issued by October 6. As an interim conservative measure, reactor power has been reduced 2.5%.

In parallel with the testing effort, the Supply System is investigating instrument and plant computer calibration history and evaluating the potential nozzle degradation which would affect feedwater flow.

At the time of discovery, evaluation verified that thermal limit margins were greater than 10%. Although License Core Thermal Power may have been exceeded, it appears that there are no immediate reactor safety issues.

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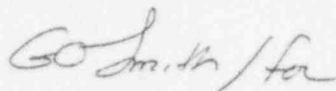
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**FACILITY OPERATION AT REACTOR CORE POWER LEVELS
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An Incident Review Board has been convened to investigate this event. Consistent with the requirements of license condition 2.F, the Supply System will submit a written followup report within 14 days.

Should you have any questions or desire additional information, please call D. A. Swank at (509) 377-4563.

Sincerely,



J. V. Parrish (Mail Drop 1023)
Vice President, Nuclear Operations

JVP/LCF/ml

cc: Document Control Desk - NRC
KE Perkins, Jr. - NRC RIV, Walnut Creek Field Office
NS Reynolds - Winston & Strawn
JW Clifford - NRC
DL Williams - BPA/399
NRC Sr. Resident Inspector - 927N