

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Millstone Point Unit 2 DOCKET NUMBER (2) 0 5 0 0 0 3 3 6 PAGE (3) 1 OF 0 2

TITLE (4) Fire Barrier Violation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)															
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)														
d	7	0	3	8	4	8	4	0	0	9	0	0	0	7	3	0	8	4	N/A	0	5	0	0	0

OPERATING CODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
POWER LEVEL (10)	0	7	5	20.402(b)	20.406(c)	50.73(a)(2)(iv)	73.71(b)				
				20.406(a)(1)(i)	50.38(e)(1)	50.73(a)(2)(v)	73.71(c)				
				20.406(a)(1)(ii)	50.38(e)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 306A)				
				20.406(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)					
				20.406(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)					
				20.406(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)					

LICENSEE CONTACT FOR THIS LER (12)
NAME Keith D. Deslandes/Plant Engineer TELEPHONE NUMBER 210341471-117911
AREA CODE

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)									
CAUSE	SYSTEM	COMPONENT	MANUFACTURE	REPORTABLE TO NPD	CAUSE	SYSTEM	COMPONENT	MANUFACTURE	REPORTABLE TO NPD
		N/A							
		N/A							

SUPPLEMENTAL REPORT EXPECTED (14)
YES (if yes, complete EXPECTED SUBMISSION DATE) X NC
EXPECTED SUBMISSION DATE (15) MONTH DAY YEAR 0 0 0 0 0 0

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During a walk-through fire inspection on July 3, 1984 at 1400 hours, while the plant was at 75% power it was discovered that the door between the East and West 480 Volt Switchgear rooms did not have the required fire rating. Immediately after the finding a continuous fire watch was established in accordance with Technical Specification Section 3.7.10, Action a.

Subsequent to posting the fire watch, an engineering review determined the door as is, minus a U-L rating, constituted a temporary fire barrier of equal effectiveness and that a fire watch was not required. For the complete engineering review please see the text.

Corrective action taken was replacement of the door with a U-L rated 3 hour fire door.

To prevent a recurrence, Engineering personnel have been reminded that all planned changes in fire barriers must have a fire protection engineering review, before making the changes.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (5)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Millstone Point Unit 2	0 5 0 0 0 3 3 6	8 4	— 0 0 9	— 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

During a walk-through fire inspection on July 3, 1984 at 1400 hours, while the plant was at 75% power it was discovered that the door between the East and West 480 Volt Switchgear rooms did not have the required fire rating. Immediately after the finding a continuous fire watch was established in accordance with Technical Specification Section 3.7.10, Action a.

Subsequent to posting the fire watch, an engineering review determined the door as is, minus a U-L rating, constituted a temporary fire barrier of equal effectiveness and that a fire watch was not required. This determination was based on the following points:

- ° NFPA requires that hollow metal fire doors be constructed of 20 gage or heavier steel. This door is constructed of 18 gage steel skin supported by 14 gage steel ribs horizontally spaced approximately 6" on center. Steel plates of 1/8" thickness form the edges of the door.
- ° The door's manufacturer, who is U-L licensed, has informed NU that the door was manufactured using three hour rated construction practices. The only exception is the 10"x10" wire glass panel which is normally utilized in 1½ hour rated construction.
- ° The door is installed in a three hour door frame.
- ° Low fire loadings exist on each side of the barrier and they are:

Area T4-10,300 BTU's/sq. foot (approximately 8 minutes fire duration).

Area A15-40,200 BTU's/sq. foot (approximately 30 minutes fire duration).

- ° Combustibles are limited to cables which are IEEE-383 qualified.
- ° Combustibles are not located in close proximity to the door.
- ° Smoke detection exists on both sides of the barrier.
- ° The area is in close proximity to the Control Room and is patrolled, every four hours.

The unrated door was inadvertantly installed as a replacement for the original door that had binding problems and no review of fire barrier effects was done at that time.

To prevent a recurrence, Engineering personnel have been reminded that all planned changes in fire barriers must have a fire protection engineering review, before making the changes.

NORTHEAST UTILITIES



THE CONNECTICUT LIGHT AND POWER COMPANY
WESTERN MASSACHUSETTS ELECTRIC COMPANY
HOLYOKE WATER POWER COMPANY
NORTHEAST UTILITIES SERVICE COMPANY
NORTHEAST NUCLEAR ENERGY COMPANY

General Offices • Selden Street, Berlin, Connecticut

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July 30, 1984
MP-6248

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Reference: Facility Operating License No. DPR-65
Docket No. 50-336
Reportable Occurrence RO 50-336/84-009-00

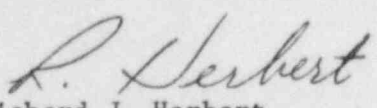
Gentlemen:

This letter forwards the Licensee Event Report 84-009-00 required to be submitted within thirty days pursuant to paragraph 50.73(a)(2)(i)(B), any operation prohibited by the plant's Technical Specification.

Yours truly,

NORTHEAST NUCLEAR ENERGY COMPANY

FOR: Edward J. Mroczka
Station Superintendent
Millstone Nuclear Power Station

BY: 
Richard J. Herbert
Station Services Superintendent
Millstone Nuclear Power Station

EJM/KDD:mo

Attachment: LER RO 50-336/84-009-00

cc: Dr. T. E. Murley, Region I

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
			Sequential	Revision	
		Year	Number	Number	
Arkansas Nuclear One, Unit 2	10510101 31 61 81	81 41	-- 0 1 3	-- 0 0	10210F1012

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 6/17/84, at 1220 hours moderator temperature coefficient (MTC) testing which is conducted at approximately 95% FP, was completed and a power increase to 100% FP was commenced. 100% FP was attained at 1243 hours. Control element assembly (CEA) bank 6 was being used to stabilize reactor power when CEA #1 (EIIIS Identifier = 02AA-Rod-001) dropped into the core. As a result, penalty factors used in the Core Protection Calculators (CPC) calculations generated a low DNBR value and a reactor trip occurred at 1249 hours. CEA #1 is located in the center location of the core, therefore, no tilt problems occurred with insertion of this CEA. During the automatic transfer of auxiliary power from the unit auxiliary transformer to startup transformer #3, 6900 volt bus 2H2 lockout relay tripped, preventing startup transformer (SU) #3 feeder breaker to 2H2 bus from closing and resulting in the bus being de-energized. As a result Reactor Coolant Pumps (RCP) 2P-32B and 2P-32C tripped as well as circulating water pump 2P-3B. 2P-32B and 2P-32C were restarted at 1456 hours and 1535 hours respectively. No post-trip difficulties were noted as a result of the loss of the 2H2 bus. Forced circulation was maintained with one pump per loop as bus 2H1 remained energized and provided power for RCP's 2P-32A and 2P-32D. The emergency feedwater system actuated as required. Manual control was taken of the emergency feedwater system to maintain the desired steam generator levels. Post-trip parameters were acceptable and posed no operational problems. Troubleshooting revealed no definitive cause for the dropped CEA. Visicorder traces of CEA #1 were normal. Withdrawal and insertion of CEA #1 during troubleshooting yielded no malfunctions. The cause may have been due to a sluggish upper gripper coil, but this could not be confirmed. Additionally, a CEA exercise was performed on 6/8/84, with satisfactory results.



ARKANSAS POWER & LIGHT COMPANY

POST OFFICE BOX 551 LITTLE ROCK, ARKANSAS 72203 (501) 371-4000

July 17, 1984

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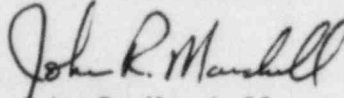
U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Arkansas Nuclear One - Unit 2
Docket No. 50-368
License No. NPF-6
Licensee Event Report
No. 84-013-00

Gentlemen:

In accordance with 10CFR50.73(a)(2)(iv), attached is the subject report concerning a reactor trip generated by the Core Protection Calculators.

Very truly yours,


John R. Marshall
Manager, Licensing

JRM:RJS:ac

Attachment

cc: Mr. Richard C. DeYoung
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, DC 20555

Mr. Norman M. Haller, Director
Office of Management & Program Analysis
U. S. Nuclear Regulatory Commission
Washington, DC 20555

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