



**Commonwealth Edison**

One First National Plaza, Chicago, Illinois  
Address Reply to: Post Office Box 767  
Chicago, Illinois 60690

July 25, 1984

Mr. Harold R. Denton, Director  
Office of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555

Subject: LaSalle County Station Units 1 and 2  
Proposed Amendment to Technical  
Specification for Facility Operating  
License NPF-11 and NPF-18 Concerning  
Accident Monitoring Instrumentation  
NRC Docket Nos. 50-373 and 50-374

- References (a): Technical Specification 3.3.7.5 (pages  
3/4 3-69 and 3/4 3-70).
- (b): Generic Letter 83-36, dated November 1,  
1984. NUREG-0737 Technical Specifications.
- (c): FSAR Section L. 30 Accident Monitoring  
Instrumentation.
- (d): SER II.F.1 Attachment 1 (pages 23-71 through  
23-77), Attachment 3 (pages 22-82 through  
22-84) and Attachment 6 (page 22-87).
- (e): SSER No. 2 II.F.1 (page 22-9).

Dear Mr. Denton:

Pursuant to 10 CFR 50.59, Commonwealth Edison proposes to amend  
Appendix A, Technical Specification, to Facility Operating Licenses NPF-11  
and NPF-18. These amendment changes are being submitted for your  
staff's review and approval.

The proposed change is enclosed in Attachment 2. The attached  
change has received both On-Site and Off-Site review and approval. We  
have reviewed this amendment request and find that no significant hazards  
consideration exists. Our review is documented in Attachment 3.  
Commonwealth Edison is notifying the State of Illinois of our request for  
this amendment by transmitting a copy of this letter and its attachments  
to the designated State Official.

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PDR ADOCK 05000373  
P PDR

Rec'd W/CHUCK \$150.00

H. R. Denton

- 2 -

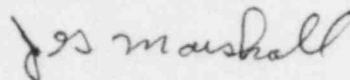
July 25, 1984

In accordance with 10 CFR 170, a fee remittance of \$150.00 is enclosed.

Please direct any questions you may have concerning this matter to this office.

Three (3) signed originals and thirty-seven (37) copies of this transmittal and its attachments are provided for your use.

Very truly yours,

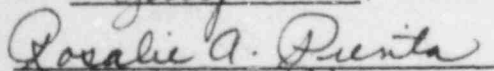


J. G. Marshall  
Nuclear Licensing Administrator

Attachments (1): Background  
(2): Technical Specification Change to NFF-18  
(3): Evaluation of Significant Hazards Consideration

cc: Region III Inspector - LaSalle  
A. Bournia - NRR  
G. Wright - Ill.

SUBSCRIBED and SWORN to  
before me this 25th day  
of July, 1984

  
Notary Public

9010N

## ATTACHMENT 1

### LASALLE COUNTY STATION UNITS 1 AND 2 TECHNICAL SPECIFICATION CHANGE REQUEST

BACKGROUND: The LaSalle County Station Unit 1 and Unit 2 Technical Specifications (Appendix A to Licenses NPF-11 and NPF-18) contain Accident Monitoring Instrumentation Limiting Conditions for Operation for Noble Gas Effluent Monitors (II.F.1.1), Containment High-Range Radiation Monitors (II.F.1.3) and Containment Hydrogen Monitor (II.F.1.6) as required by reference b).

During the review of reference b) it was noted that the present LaSalle County Station Technical Specifications (reference a) were more limiting than those proposed in Enclosure 3 of Reference b). The changes to LaSalle County Units 1 and 2 Technical Specification 3.3.7.5 as indicated on Attachment A reflect the model Technical Specifications of reference b)

#### DISCUSSION:

LaSalle County's compliance with NUREG-0737 for items II.F.1.1, II.F.1.3 and II.F.1.6 are discussed in references c), d), e). As stated in reference b) the Technical Specifications should be revised for the following reasons:

II.F.1.1: Noble Gas effluent monitors provide information, during and following an accident, which are considered helpful to the operator in accessing the plant condition. It is desired that these monitors be operable at all times during plant operation, but they are not required for safe shutdown of the plant. In case of failure of the monitor, appropriate actions should be taken to restore its operational capability in a reasonable period of time. Considering the importance of the availability of the equipment and possible delays involved in administrative controls, 7 days is considered to be the appropriate time period to restore the operability of the monitor. An alternate method for monitoring the effluent should be initiated as soon as practical, but no later than 72 hours after the identification of the failure of the monitor. If the monitor is not restored to operable condition within 7 days after the failure, a special report should be submitted to the NRC within 14 days following the event, outlining the cause of inoperability, actions taken and the planned schedule for restoring the system to operable status.

- II.F.1.3 A minimum of two in containment radiation-level monitors with a maximum range of  $10^8$  rad/hr ( $10^7$  r/hr for photon only) should be operable at all times except for cold shutdown and refueling outages. In case of failure of the monitor, appropriate actions should be taken to restore its operational capability as soon as possible. If the monitor is not restored to operable condition within 7 days after the failure, a special report should be submitted to the NRC within 14 days following the event, outlining the cause of inoperability, actions taken and the planned schedule for restoring the equipment to operable status.
- II.F.1.6 The containment hydrogen concentration analyzers monitor the primary containment gas for hydrogen. These monitors are normally in standby and initiate on reactor water level low (level 2), or high drywell pressure. The primary containment at LaSalle County Station is inerted with nitrogen per Technical Specification and has 2 redundant hydrogen recombiners (Technical Specification 3.6.6.1)

Since the inoperability of the hydrogen monitors is lesser concern than other accident monitoring instrumentation due

Printout cancelled by operator.