

LICENSEE EVENT REPORT (LER)

| | | |
|--|--------------------------------------|--------------------|
| FACILITY NAME (1) Callaway Plant Unit 1 | DOCKET NUMBER (2) 0 5 0 0 0 4 8 3 | PAGE (3) 1 OF 3 |
|--|--------------------------------------|--------------------|

| |
|---|
| TITLE (4) Steamline Low Pressure Channels Inoperable |
|---|

| EVENT DATE (6) | | | LER NUMBER (8) | | | REPORT DATE (7) | | | OTHER FACILITIES INVOLVED (9) | | |
|----------------|-----|------|----------------|-------------------|-----------------|-----------------|-----|------|-------------------------------|--|------------------|
| MONTH | DAY | YEAR | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | MONTH | DAY | YEAR | FACILITY NAMES | | DOCKET NUMBER(S) |
| 0 | 9 | 25 | 84 | 04 | 3 | 0 | 0 | 10 | | | 0 5 0 0 0 |
| | | | | | | | | | | | 0 5 0 0 0 |

| | | | | | | | | | | | |
|-------------------------|--|--|------------------|------------------|--|-----------------|----------------------|--|--|--|--|
| OPERATING MODE (9) 3 | THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11) | | | | | | | | | | |
| | 20.402(b) | | | 20.405(e) | | | 50.73(a)(2)(iv) | | | 73.71(b) | |
| | 20.405(a)(1)(i) | | | 50.36(c)(1) | | | X 50.73(a)(2)(v) | | | 73.71(c) | |
| | 20.405(a)(1)(ii) | | | 50.36(c)(2) | | | 50.73(a)(2)(vii) | | | OTHER (Specify in Abstract below and in Text, NRC Form 366A) | |
| | 20.405(a)(1)(iii) | | | X 50.73(a)(2)(i) | | | 50.73(a)(2)(viii)(A) | | | | |
| | 20.405(a)(1)(iv) | | | 50.73(a)(2)(ii) | | | 50.73(a)(2)(viii)(B) | | | | |
| 20.405(a)(1)(v) | | | 50.73(a)(2)(iii) | | | 50.73(a)(2)(ix) | | | | | |

| | | | | | | | | | |
|--|--|--|--|--|--|--|--|--------------------|--|
| LICENSEE CONTACT FOR THIS LER (12) | | | | | | | | | |
| NAME Charles D. Naslund - Superintendent, I&C | | | | | | | | TELEPHONE NUMBER | |
| | | | | | | | | AREA CODE 3 1 4 | |
| | | | | | | | | 6 7 6 - 8 5 0 0 | |

| COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) | | | | | | | | | |
|--|--------|-----------|--------------|-------------------|-------|--------|-----------|--------------|-------------------|
| CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NRC | CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NRC |
| | | | | | | | | | |
| | | | | | | | | | |
| | | | | | | | | | |

| | | | | | | | | | |
|--|--|--|--|--|--|-------------------------------|-------|-----|------|
| SUPPLEMENTAL REPORT EXPECTED (14) | | | | | | EXPECTED SUBMISSION DATE (15) | MONTH | DAY | YEAR |
| YES (If yes, complete EXPECTED SUBMISSION DATE) X NO | | | | | | | | | |

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 9/25/84 it was determined that the lead-lag controllers in the steamline pressure instrumentation loops had been incorrectly calibrated. The four STEAMLINE LOW PRESSURE-SAFETY INJECTION trips were then declared inoperable per Technical Specification 3.3.2. These trips had been required to be operable since 9/6/84 when RCS pressure went above 1970 psig.

It was found that incorrect nomenclature had been used in a generic calibration procedure, causing setpoints on each lead-lag controller to be incorrectly calibrated.

To comply with Technical Specification 3.3.2, RCS pressure was reduced to 1800 psig. The incorrect procedure was revised and the affected loops recalibrated. Additionally, all instrumentation loops which had been required to be operable were examined for similar errors. No other instrumentation loops were affected by this incident.

3411060569 841025
PDR ADOCK 05000483
S PDR

IE 22
11

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 7150-0104

EXPIRES: 8/31/85

| | | | | | | |
|--|-------------------------------------|----------------|-------------------|-----------------|----------|-------|
| FACILITY NAME (1) Callaway Plant Unit 1 | DOCKET NUMBER (2) 0500048384 | LER NUMBER (6) | | | PAGE (3) | |
| | | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | | |
| | | | 043 | 000 | 2 | OF 03 |

TEXT (If more space is required, use additional NRC Form 365A's) (17)

On 9/25/84 the plant was in Mode 3 with Reactor Coolant System (RCS) temperature and pressure 557°F and 2235 psig, respectively. Surveillance procedure ISL-BB-OT431, "LOOP-TEMP: LOOP 3 $\Delta T/TAVG$," was being performed as required per Technical Specifications prior to entry into Mode 2 and initial criticality. During the course of this procedure it was discovered that generic calibration procedure ITG-ZZ-WNLL1, "Westinghouse 7300 LEAD/LAG CARD," had a nomenclature error which caused the lead-lag time constants of these circuit cards to be calibrated incorrectly.

A review of instrumentation loops that utilize the lead-lag controller cards was immediately initiated. It was discovered that the twelve STEAMLINE-LOW PRESSURE loops, which had been required to be operable prior to exceeding an RCS pressure of 1970 psig, had been incorrectly calibrated as a result of the procedure error. Therefore, at 0915 CDT the four STEAMLINE LOW PRESSURE-SAFETY INJECTION trips were declared inoperable. The review revealed that no other instrumentation loops that had been required to be operable were affected by this error.

Per Technical Specifications the lead time constant of these lead-lag controllers shall be set to ≥ 50 seconds and the lag time constant set to ≤ 5 seconds. A note in the generic procedure stated that the "T1" time constant setting on the lead-lag card corresponds to the lag time constant. However, for a lead-lag function, "T1" actually sets the lead time constant. As a result of this procedure error the lead time constant had been actually set to 5 seconds and the lag time constant set to 0.5 seconds, which is in violation of Technical Specifications.

To comply with Technical Specifications 3.3.2-1.e and 3.3.2-4.d, RCS pressure was reduced below the P-11 setpoint (1970 psig) and stabilized at 1800 psig.

The generic calibration procedure, ITG-ZZ-WNLL1, has been revised to ensure that the lead-lag time constants are set in accordance with Technical Specifications. The corrected procedure was reissued 9/26/84. The affected steamline pressure loops were recalibrated and restored to service by 9/28/84. Channel calibrations of the RCS $\Delta T/TAVG$ loops were satisfactorily completed by 9/30/84 incorporating the revised generic procedure.

The purpose of the STEAMLINE LOW PRESSURE channels is to provide Safety Injection System (SIS) actuation in the event of a main steam system piping failure. Other functions that also provide SIS actuation in the event of a major steamline rupture are:

1. Four PRESSURIZER LOW PRESSURE channels
2. Three HIGH-1 CONTAINMENT PRESSURE channels

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 3/31/85

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (6)

PAGE (3)

Callaway Plant Unit 1

YEAR SEQUENTIAL REVISION

NUMBER NUMBER NUMBER

0 5 0 0 0 4 8 3 8 4 - 0 4 3 - 0 0 0 3 OF 0 3

TEXT (If more space is required, use additional NRC Form 365A's) (17)

As these functions were not affected by the incorrect calibrations and are also required to be operable in conjunction with the STEAMLINE LOW PRESSURE channels, there was no effect on public health and safety.

Previous occurrences: none

UNION ELECTRIC COMPANY
CALLAWAY PLANT

MAILING ADDRESS:
P.O. BOX 620
FULTON, MO. 65251

October 25, 1984

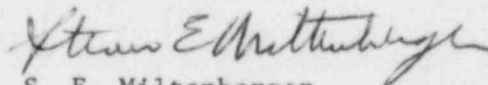
U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

ULNRC-956

DOCKET NUMBER 50-483
CALLAWAY PLANT UNIT 1
FACILITY OPERATING LICENSE NPF-30
LICENSEE EVENT REPORT 84-043-00
STEAMLINE LOW PRESSURE CHANNELS INOPERABLE

Gentlemen:

The enclosed Licensee Event Report is submitted pursuant to 10 CFR 50.73(a)(2)(i) and 50.73(a)(2)(v) concerning incorrect calibration of the Steamline Low Pressure channels.


S. E. Miltenberger
Manager, Callaway Plant

CDN/WRR/JMS/drs
Enclosure

cc: Distribution attached

IE22
11

cc distribution for ULNRC-956

Mr. James G. Keppler
Regional Administrator
Office of Inspection & Enforcement
U.S. Nuclear Regulatory Commission
Region III
799 Roosevelt Road
Glen Ellyn, IL 60137

American Nuclear Insurers
c/o Dottie Sherman, Library
The Exchange Suite 245
270 Farmington Avenue
Farmington, CT 06032

Records Center
Institute of Nuclear Power Operations
Suite 1500
1100 Circle 75 Parkway
Atlanta, GA 30339

NRC Resident Inspector
Missouri Public Service Commission
D. F. Schnell
J. F. McLaughlin
J. E. Davis (Z40LER)
D. W. Capone/R. P. Wendling
F. D. Field
R. L. Powers
A. C. Passwater/D. E. Shafer/D. J. Walker
G. A. Hughes
W. R. Robinson (QA Record)
C. D. Naslund
R. A. McAleenan
L. K. Robertson (470) (NSRB)
Merlin Williams, Wolf Creek
SEM Chrono
3456-0021.6
3456-0260
Z40ULNRC
G56.37
N. Date