

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Browns Ferry - Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 2 5 9										PAGE (3) 1 OF 0 2																					
TITLE (4) Reactor Manual Scram due to Main Steam Relief Valve Lifting																																									
EVENT DATE (6)						LER NUMBER (6)						REPORT DATE (7)						OTHER FACILITIES INVOLVED (8)																							
MONTH		DAY		YEAR		YEAR		SEQUENTIAL NUMBER		REVISION NUMBER		MONTH		DAY		YEAR		FACILITY NAMES																							
																		DOCKET NUMBER(S)																							
																		0 5 0 0 0																							
6		2		7		8		4		8		4		0		2		7		0		0		0		7		2		4		8		4		0 5 0 0 0					
OPERATING MODE (9)				N				THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																																	
POWER LEVEL (10)				0 0 1				20.402(b)						20.406(c)						X 50.73(a)(2)(iv)						73.71(a)															
								20.408(a)(1)(i)						50.36(c)(1)						50.73(a)(2)(v)						73.71(c)															
								20.408(a)(1)(ii)						50.36(c)(2)						50.73(a)(2)(vii)						OTHER (Specify in Abstract below and in Text, NRC Form 366A)															
								20.408(a)(1)(iii)						50.73(a)(2)(i)						50.73(a)(2)(viii)(A)																					
								20.408(a)(1)(iv)						50.73(a)(2)(ii)						50.73(a)(2)(viii)(B)																					
								20.408(a)(1)(v)						50.73(a)(2)(iii)						50.73(a)(2)(ix)																					
LICENSEE CONTACT FOR THIS LER (12)																																									
NAME D. L. Smith												TELEPHONE NUMBER AREA CODE 2 0 5 7 2 9 - 0 8 6 5																													
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																									
CAUSE		SYSTEM		COMPONENT		MANUFACTURER		REPORTABLE TO NPS				CAUSE		SYSTEM		COMPONENT		MANUFACTURER		REPORTABLE TO NPS																					
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)				MONTH		DAY		YEAR																					
X YES (If yes, complete EXPECTED SUBMISSION DATE)																NO		0 9		0 5		8 4																			
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)																																									
<p>During normal unit startup following a short unit outage, at approximately 1 percent power and rated pressure, a main steam relief valve began to unseat. It continued to leak causing the torus temperature to approach Technical Specification limits and the unit was manually scrambled at 350 psig. The relief valve subsequently reseated at 100 psig. No unusual occurrences followed.</p> <p>The valve was replaced and the unit restarted. The failed valve is being disassembled with the results of the investigation to be provided in a followup report.</p>																																									
8408060113 840724 PDR ADOCK 05000259 S PDR																																									

IE22  
111

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (5)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Browns Ferry - Unit 1	0 5 0 0 0 2 5 9 8 4	—	0 2 7	—	0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 366A-2 (17))

On June 27, 1984, unit 1 was in startup at 1 percent power, unit 2 was at 49 percent power, and unit 3 in a refueling outage. Unit 1 was the only unit affected by this event.

Unit 1 was in startup and approaching 1 percent power at rated pressure, when the pressure control valve 1-4 (RV) unseated. As reactor (RCT) pressure decreased, torus (BT) temperature was approaching its Technical Specification limit; therefore, a reactor manual scram was initiated at 350 psig. The valve subsequently reseated at 100 psig. No unusual events followed, and there was no safety significance to this event.

The failed valve was replaced and taken to Wyle Laboratories for testing and disassembly. The results will be provided as a followup report.

Responsible Plant Section - N/A

Previous Similar Events

BFRO-50-259/7726  
BFRO-50-259/7713  
BFRO-50-260/7503  
BFRO-50-260/7430  
BFRO-50-260/7429  
BFRO-50-259/7349  
BFRO-50-259/7314

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

Browns Ferry Nuclear Plant

P. O. Box 2000

Decatur, Alabama 35602

JUL 24 1984

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D. C. 20555

Dear Sir:

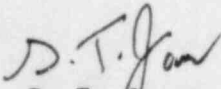
TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 1 - DOCKET  
NO. 50-259 - FACILITY OPERATING LICENSE DPR-33 - REPORTABLE OCCURRENCE  
REPORT BFRO-50-259/84027

The enclosed report provides details concerning reactor manual scram due to  
main steam relief valve lifting. This report is submitted in accordance  
with 10 CFR 50.73 (a)(2)(iv).

A 10 CFR Part 21 determination will be made following detailed disassembly  
report from Wyle Labs.

Very truly yours,

TENNESSEE VALLEY AUTHORITY



G. T. Jones  
Plant Manager  
Browns Ferry Nuclear Plant

Enclosure

cc (Enclosure):  
Regional Administrator  
U. S. Nuclear Regulatory Commission  
Office of Inspection and Enforcement  
Region II  
101 Marietta Street, Suite 2900  
Atlanta, GA 30303

INPO Records Center  
Suite 1500  
1100 Circle 75 Parkway  
Atlanta, GA 30339

NRC Resident Inspector, BFN

IE22  
11