NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
LaSalle County Station Unit 2	0 5 0 0 0 3 7 4	1 OF 0 3

TITLE (4)

Scram on Reactor Vessel High Pressure

EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER		REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
										NA	0 5 0 0 0
0 7	0 9	8 4	8 4	— 0 3 5		— 0 0	0 7	2 4	8 4		0 5 0 0 0

OPERATING MODE (9)	1
POWER LEVEL (10)	0 6 8

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

	20.402(b)		20.406(c)	X	50.73(a)(2)(iv)
	20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)
	20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)
	20.405(a)(1)(iii)		50.73(a)(2)(i)		50.73(a)(2)(viii)(A)
	20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)
	20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)

	73.71(b)
	73.71(c)
	OTHER (Specify in Abstract below and in Text, NRC Form 366A)

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Tom Hammerich, extension 259	AREA CODE 8 1 5 3 5 7 - 6 7 6 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
A	J J	Z 9 9 9	Z 9 9 9	N							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

While troubleshooting a problem with the signal from the Electrohydraulic Control (EHC) system to the Reactor Recirculating system, as part of the Startup Test Program, an Instrument Technician's probe slipped causing a momentary loss of the 30 volt power supply to the EHC system. This caused the reactor to scram on high pressure due to cycling of the main turbine bypass, control and intermediate stop valves. No ECCS actions were required and reactor pressure and level were stabilized within normal limits.

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

I. EVENT DESCRIPTION

On July 9, 1984, at 1907 with Unit 2 at 68% power, the reactor scramed on a Reactor Vessel High Pressure signal (1043 psig). At the time of the event the Instrument Maintenance Department was troubleshooting a problem with the signal from the Electrohydraulic Control System (EHC, JJ) to the Reactor Recirculation (RR, AD) Master Flow Controller. This troubleshooting had been progressing for most of the day previous to the event. The unit is in the initial Startup Testing phase.

II. CAUSE

The high pressure transient was caused by the momentary loss of the positive 30 volt power supply in the EHC cabinet. This was due to an error by the Instrument Technician who was troubleshooting the problem described above. Even though the connections which were required for the work being done were in close quarters, the technician had sucessfully performed the connections several times previously but on this occasion the probe slipped causing the momentary short to ground. This caused the Main Turbine (TA) bypass valves to cycle open and closed and at the same time the Main Turbine control and intercept valves to close which resulted in increased reactor pressure and neutron flux. As a result of the event, neutron flux increased to approximately 110% and reactor pressure increased to approximately 1080 psig as read from the Control Room indication. After the scram reactor level decreased to approximately minus 30 inches but was immediately restored to normal.

III. PROBABLE CONSEQUENCES OF THE OCCURRENCE

The reactor safety systems performed as expected. Three (3) Safety/Relief valves (U, S, and D) lifted (SRV, SB) which are those expected based on the previous calibration data for the pressure switches which open these valves. By this data, reactor pressure may have reached 1100 psig (setpoint of the D SRV) before the open SRV's returned pressure to normal. The three SRV's only opened once and the last one closed approximately 30 seconds after opening. No ECCS systems were required and reactor vessel level was immediately restored to normal. Had the reactor been operating at higher power levels the transient would not be expected to be any more severe and the scram would probably have occurred due to APRM (IG) high flux instead of high reactor pressure.

IV. CORRECTIVE ACTIONS

The troubleshooting was immediately halted. Since this particular type of test must be performed while the turbine is operating, subsequent testing of this circuit will be performed at a lower power level. No other corrective actions are required.

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

V. PREVIOUS OCCURRENCES

No high reactor pressure scrams have occurred to date. The loss of the positive 30 volt power supply of EHC has also not occurred previously.

VI. NAME AND TELEPHONE NUMBER OF PREPARER

Tom Hammerich, (815)357-6761, extension 259.

Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

July 24, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

Reportable Occurrence Report #84-035-00, Docket #050-374 is being submitted to your office in accordance with 10 CFR 50.73.

G. J. Diederich
Superintendent
LaSalle County Station

GJD/MLD/kg

Enclosure

xc: NRC, Regional Director
INPO-Records Center
File/NRC

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