

Commonwealth Edison Company
LaSalle Generating Station
2601 North 21st Road
Marseilles, IL 61341-9757
Tel 815-357-6761



October 24, 1995

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

Licensee Event Report #95-016-00, Docket #050-373 is being submitted to your office in accordance with 10CFR50.73(a)(2)(iv).

Sincerely,

A handwritten signature in dark ink, appearing to read "D. J. Ray", is written over the typed name.

D. J. Ray
Station Manager
LaSalle County Station

DJR/SW/lja

Enclosure

cc: H. J. Miller, NRC Region III Administrator
P. G. Brochman, NRC Senior Resident Inspector
R. J. Zuffa, IDNS Resident Inspector
F. Niziolek, IDNS Senior Reactor Analyst
INPO - Records Center
D. L. Farrar, Nuclear Regulatory Services Manager

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LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) LaSalle County Station Unit One	DOCKET NUMBER (2) 0500037	PAGE (3) 1 of 4
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TITLE (4) Unit One Manual Reactor Scram due to Emergency Governor Lockout Valve Sticking

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
09	24	95	95	016	00	10	24	95	None	
									FACILITY NAME	DOCKET NUMBER

OPERATING	N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)								
		20.2201(b)		20.2203(a)(3)(i)		50.73(a)(2)(iii)		73.71(b)		
POWER	000	20.2203(a)(1)		20.2203(a)(3)(ii)	x	50.73(a)(2)(iv)		73.71(c)		
		20.2203(a)(2)(i)		20.2203(a)(4)		50.73(a)(2)(v)		OTHER		
		20.2203(a)(2)(ii)		50.36(c)(1)		50.73(a)(2)(vii)		(Specify in		
		20.2203(a)(2)(iii)		50.36(c)(2)		50.73(a)(2)(viii)(A)				
		20.2203(a)(2)(iv)		50.73(a)(2)(i)		50.73(a)(2)(viii)(B)				
		20.2203(a)(2)(v)		50.73(a)(2)(ii)		50.73(a)(2)(x)				

LICENSEE CONTACT FOR THIS LER (12)

NAME Steven Wix, System Engineer	TELEPHONE NUMBER (Include Area Code) (815) 357-6761 x 2571
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS

SUPPLEMENTAL REPORT EXPECTED (14)

X YES (If yes, complete EXPECTED SUBMISSION DATE)	NO	EXPECTED	MONTH	DAY	YEAR
			3	15	95

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines 16)

At 0400 hours on September 24, 1995, Unit 1 was in Operational Condition 1 (Run) and at 100% power. Surveillance LOS-FW-SR1, "Turbine Feedwater Pump Surveillance", had begun on the 1B Turbine Driven Reactor Feed Pump (TDRFP). The 1B emergency governor hand switch was taken to the locked out position. When the "trip test" push-button was depressed the 1B TDRFP unexpectedly tripped out and reactor water level started to decrease. In response to this the Unit NSO began to manually start the 1C Motor Driven Reactor Feed Pump (MDRFP), while a third operator began reducing reactor recirculation flow via the flow control valves.

Reactor level began to recover and during this recovery the 1B TDRFP unexpectedly returned to operation. With the three feed pumps feeding reactor level, the 55.5 inch High Reactor Water Level/Main Turbine Trip was quickly reached. The control room operators initiated a manual scram just before an automatic scram would have occurred due to the turbine trip. All systems functioned as expected and the reactor safely shutdown.

The source of the failure was determined to be the emergency governor lockout valve sticking. A specific root cause for this failure could not be determined but the probable cause was foreign material intrusion from the turbine lube oil system.

LICENSEE EVENT REPORT (LER)

TEXT CONTINUATION

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If more space is required, use additional copies of NRC Form 366A (17)

PLANT AND SYSTEM IDENTIFICATION

General Electric - Boiling Water Reactor

Energy Industry Identification System (EIIS) codes are identified in the text as [XX].

A. CONDITION PRIOR TO EVENT

Unit(s): 1 Event Date: 9/24/95 Event Time: 0408 Hours

Reactor Mode(s): 1 Modes(s) Name: Run Power Level(s): 100%

B. DESCRIPTION OF EVENT

At 0400 hrs on September 24, 1995, Unit 1 was in Operational Condition 1 (Run) and at 100% power. Surveillance LOS-FW-SR1 "Turbine Feedwater Pump Surveillance", had been performed successfully on the 1A Turbine Driven Reactor Feed Pump (TDRFP,FW)(SJ) and had begun on the 1B TDRFP. The 1B TDRFP thrust bearing checks portion of LOS-FW-SR1 were completed satisfactory and the emergency governor lockout testing commenced. The 1B TDRFP emergency governor handswitch was taken to the "locked out" position and held while checks were performed for the proper indications at the control room panel. All indications were "as required" and the "trip test" push-button was then depressed. Unexpectedly the 1B TDRFP flow indication dropped to zero and the "Trip" light went on indicating that the TDRFP actually tripped. The Unit Assist Nuclear Station Operator(NSO) realizing that a failure in the trip bypass had occurred, made multiple attempts to reset the 1B TDRFP. Reactor water level started to decrease. In response to this, the Unit NSO manually started the 1C Motor Driven Reactor Feed Pump(MDRFP) while a third operator reduced reactor recirculation flow via the flow control valves. Preparation for manually scrambling the unit prior to reaching the Low Level trip was initiated but was aborted as vessel level began to increase. A reactor "Low Level" scram was avoided by these actions.

The 1B TDRFP, while still in manual mode, unexpectedly returned to operation and began adding to the reactor water inventory. The reactor recirculation pumps downshifted from fast to slow speed and with the three feed pumps feeding reactor level, the 55.5 inch High Reactor Water Level/Main Turbine Trip was quickly reached. A manual reactor scram was initiated after the Reactor High Water Level 55.5 inch trip (Loss of Main Turbine) occurred. All feedwater pumps (1A and B TDRFP and 1C MDRFP) tripped off as required. All systems functioned as expected and reactor shutdown and recovery was completed satisfactorily.

A formal root cause was performed to determine the problem(s) that may have caused the failure of the emergency governor lockout trip test.

This event is reportable pursuant to 10 CFR 50.73(a)(2)(iv)

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TEXT CONTINUATION

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C. CAUSE OF EVENT

Testing began with a non-intrusive inspection to determine if any problems existed prior to disassembly. This included electrical testing on selected relay solenoids, pressure switch calibration verification, oil analysis, and a visual inspection of control linkages and components. Complete disassembly of the front standard then began and the only irregularity found was that the emergency governor lockout valve was binding in its cylinder. The lockout valve cylinder was lightly honed for removal of small burrs, a new valve spool was installed and tested for freedom of movement (rotation and stroke), and then the 1B TDRFP was reassembled. During subsequent feedwater pump testing, the emergency governor trip test was completed satisfactorily but the TDRFP failed to trip from the control room when the turbine trip push button was depressed. This failure indicated a possible problem at the trip dump valve assembly.

The feedpump was taken out of service (OOS) and a detailed component inspection and analysis of foreign material performed. No specific root cause for the failure could be found, but it was determined that the trip dump valve bound mechanically with a particle or particles within the trip dump valve assembly. By design, a large amount of oil passes through this pathway, therefore when the trip dump valve movement was freed and this component was inspected no particulate matter was found.

The probable root cause has been determined to be foreign material from the Turbine Lube Oil system(TO)(TD). Testing of lube oil samples taken from the affected components of the hydraulic control system have indicated that a crud burst of material may have been carried to the hydraulic controls during the change from the Auxiliary motor driven oil pump to the Main Turbine Main Shaft Oil Pump (MSOP) when the main turbine is initially brought on line. We are currently evaluating the installation of an in-line filter for the control oil system

D. ASSESSMENT OF SAFETY CONSEQUENCES

The safety consequences of this event were minimal and all Engineered Safety Feature (ESF) actuations occurred as designed during this event. The potential for loss of feedwater control and increase in reactor level with the reactor at 100.0% power is an analyzed condition of moderate frequency (Updated Final Safety Analysis Report, Section 15.1.2A, "Loss of Feedwater Controller Failure Maximum Demand"). If a TDRFP mechanical trip mechanism fails to close the TDRFP stop valves, the TDRFP speed control system will close the control valves on a reactor vessel level 8 trip signal. This control valve closure will stop TDRFP flow to the reactor pressure vessel(RPV). A manual reactor scram was initiated after the Reactor High Water Level 55.5 inch trip (Loss of Main Turbine)and all feedwater pumps (1A and B TDRFP and 1C MDRFP) tripped off as required. The reactor was safely shut down and recovery performed without incident.

E. CORRECTIVE ACTIONS

The 1B TDRFP was isolated and taken out of service to determine the cause of the emergency governor lockout valve sticking. Oil samples were taken of the supply oil (Turbine Lube Oil System) for the 1A and 1B TDRFP for analysis and the results were found within the required GE specifications. The 1A TDRFP control filters were inspected for any similar problems due to the common oil system and multiple tests(LOS-FW-SR1, "Turbine Feedwater Pump Surveillance" and LOS-BO-SR2, "Operability of Main turbine Support Systems and Turbine Driven Reactor Feedpump Trip Solenoids") of the system was performed. Additional areas checked included: electrical connections, pressure switch calibrations, linkage connections and freedom of movement. All were found satisfactory.

366A U.S. NUCLEAR REGULATORY COMMISSION		APPROVED BY OMB NO. 3150-0104 EXPIRES 5/31/95	
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The addition of in-line filtering system for the TDRFP oil system will be evaluated. A supplement to this Licensee Event Report will be submitted upon completion of this evaluation.

F. PREVIOUS OCCURRENCES

LER Number	Title
DVR #01-02-90-056	2B TDRFP failure to trip
373/94-10-00	Scram due to Reactor Level Control System Loss to 1B Turbine Driven Reactor Feed Pump
373/94-11-00	U-1 Scram due to a Feedwater Signal Spike

G. COMPONENT FAILURE DATA

Since no component failure occurred, this section is not applicable