

NRC Form 366  
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Grand Gulf Nuclear Station - Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 4 1 6	PAGE (3) 1 OF 0 2
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TITLE (4) Vessel Level Transmitters Exceed Technical Specification Limits
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EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 6	2 8	8 4	8 4	0 3 2	0 0	0 7	2 7	8 4	NA		0 5 0 0 0
<div style="display: flex; justify-content: space-between;"> <span>0 6 2 8 8 4</span> <span>0 3 2 0 0</span> <span>0 7 2 7 8 4</span> <span>0 5 0 0 0</span> </div>											

OPERATING MODE (9) 4		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11)									
POWER LEVEL (10) 0 0 0		20.402(b)		20.405(v)		50.73(a)(2)(iv)		73.71(b)			
		20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)		73.71(c)			
		20.405(a)(1)(ii)		50.36(c)(2)	X	50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)			
		20.405(a)(1)(iii)		50.73(a)(2)(i)		50.73(a)(2)(viii)(A)					
		20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)					
	20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(ix)						

LICENSEE CONTACT FOR THIS LER (12)		TELEPHONE NUMBER	
NAME Ronald W. Byrd/Licensing Engineer		AREA CODE 6 0 1 4 3 7 - 2 1 4 9	

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS		
D	J C	L T R	3 6 9								

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Reactor vessel level transmitter calibration procedures were recently revised to compensate for the effect of condensate pot movement during vessel heatup to operating temperature and to include more accurate level elevation values. When the revised calibration procedures were performed, three transmitters were found to have been outside Technical Specification limits due to the values used in the previous calibration.

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PDR ADOCK 05000416  
S PDR

NRC Form 365A  
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Grand Gulf Nuclear Station - Unit 1	0500041684	-	032	-	0002	OF	02

TEXT (If more space is required, use additional NRC Form 365A's) (17)

During a reactor vessel nuclear heatup, data was taken to determine the effect of condensate pot movement on vessel level instrumentation. As a result, vessel level instrument calibration procedures were revised using new level elevation values to calibrate transmitters for a more accurate reading at vessel operating temperatures. When the revised procedures were performed, three of the transmitters were found to be outside of Technical Specification limits due to values used in the previous calibration.

The RPS level 8 trip, Channel D, exceeded the maximum 54.1 inch limit with an as-found value of 54.8 inches. The RPS level 3 trip, Channel C, exceeded the minimum 10.8 inches with an as-found value of 9.8 inches. The level 1 trip Channel C, would have occurred at -152.6 inches rather than at the Technical Specification maximum limit of -152.5 inches.

One function of the level 1 trip is to close the Main Steam Line drain valves. The inboard drain valve isolation would have occurred 0.1 inch lower than the Technical Specification limit since both B and C Channels are required for the trip function. The outboard isolation valve would have tripped within Technical Specification limits.

All other trip functions required by these systems would have occurred within the Technical Specification limits as other channels were available to complete the logic. In addition, the Level 8 trip is not required until Operational Condition 1.

This is reported pursuant to 10CFR 50.73 (a)(2)(vii) due to the D and C channels of independent systems becoming inoperable as a result of a common cause. The procedure changes and recalibrations have been completed.



# MISSISSIPPI POWER & LIGHT COMPANY

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P. O. BOX 1640, JACKSON, MISSISSIPPI 39205

NUCLEAR LICENSING & SAFETY DEPARTMENT

July 27, 1984

Document Control Desk  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Gentlemen:

SUBJECT: Grand Gulf Nuclear Station  
Unit 1  
Docket No. 50-416  
License No. NPF-13  
File: 0260/L-835.0  
Vessel Level Transmitters Exceed  
Technical Specification Limits  
LER 84-032-0  
AECM-84/0380

Attached is Licensee Event Report (LER) 84-032-0 which is a final report.

Yours truly,

*L. F. Dale*  
L. F. Dale  
Director

EBS/SHH:rg  
Attachment

cc: Mr. J. B. Richard (w/a)  
Mr. R. B. McGehee (w/o)  
Mr. N. S. Reynolds (w/o)  
Mr. G. B. Taylor (w/o)

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