

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
Washington Nuclear Plant - Unit 2DOCKET NUMBER (2)
050003971 OF 02TITLE (4)
Containment Temperature Monitoring

| EVENT DATE (5) | | | LER NUMBER (6) | | REPORT DATE (7) | | | OTHER FACILITIES INVOLVED (8) | | |
|----------------|-----|------|----------------|-------------------|-----------------|-------|-----|-------------------------------|----------------|------------------|
| MONTH | DAY | YEAR | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | MONTH | DAY | YEAR | FACILITY NAMES | DOCKET NUMBER(S) |
| 07 | 03 | 84 | 84 | 034 | 01 | 07 | 26 | 84 | | 050003971 |

| OPERATING MODE (9) | THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 50. (Check one or more of the following) (11) | | | | | | | | | | | | | | | | | | | | | | | | |
|--------------------|---|----------------------|--|-----------------|----------|-----------------|-------------|----------------|----------|------------------|-------------|-----------------|--|-------------------|----------------|---------------------|----------------|------------------|-----------------|----------------------|---------------------|-----------------|------------------|-----------------|--|
| 1 | <table border="1"><tr><td>20.402(b)</td><td>20.408(a)</td><td>50.73(a)(2)(iv)</td><td>73.71(b)</td></tr><tr><td>20.408(a)(1)(i)</td><td>50.38(a)(1)</td><td>50.73(a)(2)(v)</td><td>73.71(c)</td></tr><tr><td>20.408(a)(1)(ii)</td><td>50.38(a)(2)</td><td>50.73(a)(2)(vi)</td><td>X OTHER (Specify in Abstract below and in Text, NRC Form 366A)</td></tr><tr><td>20.408(a)(1)(iii)</td><td>50.73(a)(2)(i)</td><td>50.73(a)(2)(vii)(A)</td><td>Special Report</td></tr><tr><td>20.408(a)(1)(iv)</td><td>50.73(a)(2)(ii)</td><td>50.73(a)(2)(viii)(B)</td><td>Tech. Spec. 3.7.8.a</td></tr><tr><td>20.408(a)(1)(v)</td><td>50.73(a)(2)(iii)</td><td>50.73(a)(2)(ix)</td><td></td></tr></table> | 20.402(b) | 20.408(a) | 50.73(a)(2)(iv) | 73.71(b) | 20.408(a)(1)(i) | 50.38(a)(1) | 50.73(a)(2)(v) | 73.71(c) | 20.408(a)(1)(ii) | 50.38(a)(2) | 50.73(a)(2)(vi) | X OTHER (Specify in Abstract below and in Text, NRC Form 366A) | 20.408(a)(1)(iii) | 50.73(a)(2)(i) | 50.73(a)(2)(vii)(A) | Special Report | 20.408(a)(1)(iv) | 50.73(a)(2)(ii) | 50.73(a)(2)(viii)(B) | Tech. Spec. 3.7.8.a | 20.408(a)(1)(v) | 50.73(a)(2)(iii) | 50.73(a)(2)(ix) | |
| 20.402(b) | 20.408(a) | 50.73(a)(2)(iv) | 73.71(b) | | | | | | | | | | | | | | | | | | | | | | |
| 20.408(a)(1)(i) | 50.38(a)(1) | 50.73(a)(2)(v) | 73.71(c) | | | | | | | | | | | | | | | | | | | | | | |
| 20.408(a)(1)(ii) | 50.38(a)(2) | 50.73(a)(2)(vi) | X OTHER (Specify in Abstract below and in Text, NRC Form 366A) | | | | | | | | | | | | | | | | | | | | | | |
| 20.408(a)(1)(iii) | 50.73(a)(2)(i) | 50.73(a)(2)(vii)(A) | Special Report | | | | | | | | | | | | | | | | | | | | | | |
| 20.408(a)(1)(iv) | 50.73(a)(2)(ii) | 50.73(a)(2)(viii)(B) | Tech. Spec. 3.7.8.a | | | | | | | | | | | | | | | | | | | | | | |
| 20.408(a)(1)(v) | 50.73(a)(2)(iii) | 50.73(a)(2)(ix) | | | | | | | | | | | | | | | | | | | | | | | |

LICENSEE CONTACT FOR THIS LER (12)
NAME
C.M. Powers, Reactor Engineering SupervisorTELEPHONE NUMBER
509 377-2501
Ext. 2996

| COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) | | | | | | | | | |
|--|--------|-----------|--------------|-------------------|-------|--------|-----------|--------------|-------------------|
| CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NRC | CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NRC |
| B | V | B | - | - | - | - | N | | |

SUPPLEMENTAL REPORT EXPECTED (14)
X YES (If yes, complete EXPECTED SUBMISSION DATE)
NO
EXPECTED SUBMISSION DATE (15)
12 01 84

ABSTRACT (Limit to 1400 spaces - e. approximately fifteen single-space typewritten lines) (16)

Revision 0 of LER 034 was submitted on May 11, 1984 describing drywell temperature indications above 150°F. This revision is being issued to report that on July 3, 1984 drywell temperature monitor CMS-TI-55 indicated above 150°F for a period of greater than 8 hours and thus, also falls into the reporting category of Technical Specification requirement 3.7.8.a. This condition is a continuation of the original problem addressed in Revision 0.

The average drywell temperature did not exceed 135°F as per Technical Specification 3.6.1.7 during the event of July 3, 1984.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

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|--|---|----------------|----------------------|--------------------|----------|--|--|
| FACILITY NAME (1) Washington Nuclear Plant - Unit 2 | DOCKET NUMBER (2) 0 5 0 0 0 3 9 7 8 4 - 0 3 4 - 0 1 0 2 OF 0 2 | LER NUMBER (6) | | | PAGE (3) | | |
| | | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | | | |
| | | | | | | | |

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Plant Condition

- a). Power Level - 35%
b). Plant Mode - 1

Event

On July 3, 1984 drywell temperature monitor CMS-TI-55 indicated temperatures above 150°F in the upper drywell area return duct. The temperature exceeded 150°F for over 8 hours and, thus is reportable per Technical Specification 3.7.8.a, although the drywell average air temperature did not exceed 135°F.

Immediate Corrective Action

This condition is considered a part of the ongoing problem identified in Rev. 0 of LER 84-034. Consequently, no additional corrective action was initiated as a result of this event.

Future Corrective Action

Modifications to the design of the Drywell H&V are being completed and field changes implemented to correct the high temperature conditions. A final report on the modifications and results achieved will be provided by December 1, 1984.

Safety Significance

The results of local high temperatures have not jeopardized Plant safety or that of the public.

Washington Public Power Supply System

P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

Docket No. 50-397
July 26, 1984

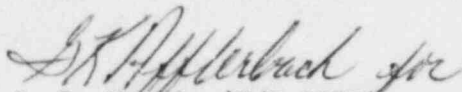
Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: NUCLEAR PLANT NO. 2
LICENSEE EVENT REPORT NO. 84-034-01

Dear Sir:

Transmitted herewith is Licensee Event Report No. 84-034-01 for WNP-2 Plant. This report is a special report and is submitted in accordance with the requirements of the WNP-2 Technical Specification Section 6.9.2.

Very truly yours,


J. D. Martin (M/D 927M)
WNP-2 Plant Manager

JDM:mm

Enclosure:
Licensee Event Report No. 84-034-01

cc: Mr. John B. Martin, Administrator
Region V, Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
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Walnut Creek, California 94596
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The Exchange Suite 245
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Farmington, CT 06032

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