

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Susquehanna Steam Electric Station-Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 3 8 8				PAGE (3) 1 OF 2											
TITLE (4) HPCI and One RHR Pump Inoperable Simultaneously.																									
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)															
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)												
0	9	1	9	8	4	8	4	0	1	9	0	0	1	0	1	8	8	4	0	5	0	0	0		
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11)																							
1		20.402(b)				20.406(e)				60.73(a)(2)(iv)				73.71(b)											
POWER LEVEL (10)		0 7 1				20.406(a)(1)(i)				60.36(e)(1)				73.71(e)											
		20.406(a)(1)(ii)				60.36(e)(2)				60.73(a)(2)(v)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)											
		20.406(a)(1)(iii)				X 60.73(a)(2)(i)				60.73(a)(2)(vii)															
		20.406(a)(1)(iv)				60.73(a)(2)(ii)				60.73(a)(2)(viii)(A)															
		20.406(a)(1)(v)				60.73(a)(2)(iii)				60.73(a)(2)(viii)(B)															
						60.73(a)(2)(ix)				60.73(a)(2)(x)															
LICENSEE CONTACT FOR THIS LER (12)																									
NAME L.A. Kuczynski, Nuclear Plant Specialist III										TELEPHONE NUMBER															
										AREA CODE 7 1 7 5 4 2 - 1 3 7 5 9															
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS															
B	BIO	BKIRW	11210	N																					
SUPPLEMENTAL REPORT EXPECTED (14)																EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR					
YES (If yes, complete EXPECTED SUBMISSION DATE)																X NO									

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On September 19, 1984, Residual Heat Removal (RHR) pump 'D' was declared inoperable due to its failure to start on a manual initiation signal. Since the High Pressure Coolant Injection (HPCI) system was also inoperable because it was undergoing a lube oil changeout, the Unit entered Limiting Condition for Operation (LCO) 3.0.3, requiring the commencement within one hour of actions to begin reactor shutdown. LCO 3.0.3 was cleared in 15 minutes when RHR pump 'D' responded properly to a manual initiation signal. The problem with the RHR pump 'D' was ultimately traced to intermittent operation of the '52LS' relay contacts on the switchgear breaker. The breaker was replaced.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1) Susquehanna Steam Electric Station Unit 2	COCKET NUMBER (2) 0 5 0 0 0 3 8 8 8 4 - 0 1 9 - 0 0 0 2 OF 0 2	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		

TEXT (if more space is required, use additional NRC Form 366A's) (17)

On September 18, 1984, at 1150, the High Pressure Coolant Injection (HPCI) system was removed from service to change its lube oil. Entry into Limiting Condition for Operation (LCO) 3.5.1 was duly noted by Operations personnel. The remaining Emergency Core Cooling Systems (Core Spray, Low Pressure Coolant Injection and Automatic Depressurization) were operable, as was the Reactor Core Isolation Cooling (RCIC) system. Thus, per the LCO Action statement, HPCI could remain inoperable for fourteen (14) days before the LCO would affect plant operation.

Upon completion of the lube oil changeout on September 19, 1984, the HPCI system was to be run to confirm its proper operation prior to clearing the LCO. Per the procedure being used to run HPCI, the Residual Heat Removal (RHR) system is to be placed in service in its suppression pool cooling mode prior to the HPCI start. (The RHR system can be lined up for any of several modes, one of which is Low Pressure Coolant Injection, an ECCS.) At 1650, the RHR pump 'D' failed to start upon a manual actuation signal. Operations personnel immediately recognized the circumstances as requiring entry into LCO 3.0.3 because the provisions of LCO 3.5.1 could no longer be met with the RHR pump inoperable. LCO 3.0.3 specifies, in effect, that when an LCO is not met except as provided in the ACTION statement, the Unit shall, within one hour, begin shutting down.

Operations personnel racked the pump's switch gear breaker out, then back in. Another attempt to start the pump was made, which was successful. LCO 3.0.3 was cleared at 1705, HPCI was run successfully and LCO 3.5.1 was cleared at 1812. Immediate investigation of the RHR pump 'D' breaker found nothing abnormal.

On October 4, 1984, there was another occurrence of RHR pump 'D' failing to start on a manual actuation signal. This time, an investigation was able to be performed before the breaker was distrubed. It was found that the '52LS' limit switch contacts were functioning intermittently. A spare breaker was functionally tested and installed to replace the breaker with the faulty limit switch. RHR pump 'D' was then test run satisfactorily.



Pennsylvania Power & Light Company

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October 18, 1984

U.S. Nuclear Regulatory Commission
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SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 84-019-00
ER 100450 FILE 841-23
PLA-2337

Docket No. 50-388
License No. NPF-22

Attached is Licensee Event Report 84-019-00. This event was determined reportable per 10CFR50.73(a)(2)(i), in that with the High Pressure Coolant Injection system out of service, one Residual Heat Removal system pump was declared inoperable and the Unit entered Limiting Condition for Operation 3.0.3.

H.W. Keiser
Superintendent of Plant-Susquehanna

LAK/pjg

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