



GULF STATES UTILITIES COMPANY

October 8 , 1984  
RBG- 19,114  
File No. G9.5,  
G9.8.6.1, G9.33.4

Mr. Harold R. Denton, Director  
Office of Nuclear Reactor Regulation  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Dear Mr. Denton:

River Bend Station-Unit 1  
Docket No. 50-458

This letter and the attached report provides the Nuclear Regulatory Commission (NRC) Staff with Gulf States Utilities Company (GSU) response to the letter from Mr. A. Schwencer (NRC) to Mr. W. J. Cahill (GSU) dated November 14, 1983. This response details GSU's position regarding the structural aspects of Safety Relief Valve (SRV) in-plant testing at River Bend Station (RBS) by addressing the five criteria set forth in Section 4 of NUREG-0763. In summary, the attached report demonstrates that the important parameters of RBS satisfy the criteria in Section 4 of NUREG-0763 and that the GESSAR load methodology (Appendix 6A of the RBS Final Safety Analysis Report) has been conservatively developed for the air bubble pressure and frequency time histories. Therefore, the test data generated from the Kuosheng Nuclear Power Station Unit 1 (Republic of China) provides the prototypical data base required to confirm the conservative nature of the SRV discharge hydrodynamic loads used in the design of River Bend Station.

The River Bend Station free-standing steel containment geometry and materials are essentially identical to those for the Perry Nuclear Power Plant Unit 1 (Cleveland Electric Illuminating Company-CEI) both of which are bounded by Kuosheng. Secondly, the results from the additional structural studies performed by CEI, using pressure time histories from the Kuosheng test as input forcing functions, are directly applicable to RBS since examination of the design response spectra for the plants have similar results in the pool region. Based upon these two conclusions (discussed in more detail in the attached report) and the precedence set by the NRC Staff for Perry, it is GSU's position that no inplant SRV discharge testing nor further modeling of design response spectra is required for River Bend Station. This

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transmittal provides the required information necessary for the NRC Staff to complete their supplemental report for Safety Evaluation Report (SER) Confirmatory Item No. 13 (Section 6.2.1.8.3, Page 6-17).

Sincerely,

*J. E. Booker*

J. E. Booker  
Manager-Engineering,  
Nuclear Fuels & Licensing  
River Bend Nuclear Group

*WJR*  
JBH/WJR/JWL/kt

Attachment