GPU Nuclear

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October 15, 1984

Mr. Walter Paulson, Acting Chief
Operating Reactors Branch No. 5
U.S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Mr. Paulson:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
SEP Topic No. III-6 Seismic Design Considerations

During the integrated assessment of the subject SEP topic, the NRC staff requested GPUN to verify the design adequacy of the piping supports for the two large piping systems analyzed by the NRC (i.e., the main steam and feed water lines).

The attached report presents the results of a seismic reanalysis of Oyster Creek's main steam and feedwater piping supports inside containment. In this report, the seismic reanalysis of main steam and feedwater piping supports inside containment was performed in accordance with the requirements of the 1980 Edition (including Winter 1982 addenda) of the ASME Code, Section III, Division 1, Subsection NF for component supports.

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As shown in the report, the piping supports meet the requirements of the 1980 Edition (including Winter 1982 Addenda) of the ASME Code confirming the adequacy of the original piping support design.

Very truly yours,

P. B. Fiedler
Vice President and Director
Oyster Creek

lr/0425e
cc: Administrator
Region I
U.S. Nuclear Regulatory Commission
7920 Norfolk Avenue
Bethesda, Md. 20014

NRC Resident Inspector
Oyster Creek Nuclear Generatng Station
Forked River, N. J. 08731