

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Dresden Nuclear Power Station										DOCKET NUMBER (2) 0 5 0 0 0 2 4 9										PAGE (3) 1 OF 2	
TITLE (4) CCSW Heat Exchanger delta P control valve inop																					
EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)												
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES					DOCKET NUMBER(S)							
0 9	0 4	8 4	8 4	0 1 4	0 0	1 0	0 4	8 4	N/A					0 5 0 0 0							
OPERATING MODE (9) N			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																		
POWER LEVEL (10) 0 8 8			20.402(b)			20.405(e)			80.736(2)(iv)			73.71(b)									
			20.405(a)(1)(i)			80.38(a)(1)			80.736(2)(v)			73.71(e)									
			20.405(a)(1)(ii)			80.38(a)(2)			80.736(2)(vi)			OTHER (Specify in Abstract below and in Text, NRC Form 305A)									
			20.405(a)(1)(iii)			80.736(2)(i)			80.736(2)(vii)(A)												
			20.405(a)(1)(iv)			80.736(2)(ii)			80.736(2)(vii)(B)												
			20.405(a)(1)(v)			80.736(2)(iii)			80.736(2)(x)												
LICENSEE CONTACT FOR THIS LER (12)												TELEPHONE NUMBER									
NAME Lawrence Coyle ext. 483												AREA CODE 8 1 5 9 4 2 - 2 9 2 0									
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																					
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS											
A				N																	
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR					
YES (If yes, complete EXPECTED SUBMISSION DATE)												X NO									

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During normal operation, LPCI/CCSW heat exchanger delta P valve MO 3-1501-3A was taken out-of-service for limit switch adjustments, rendering 'A' LPCI/CCSW loop inoperable. No ECCS surveillances were completed prior to rendering the 'A' loop inoperable (Tech. Spec. 4.5.8.3). DOS 1500-3 (Containment Cooling Pump Test), was immediately initiated to verify the operability of the 'B' LPCI/CCSW loop. The valve was returned to service after the limit switches were adjusted. Safety significance was minimal since the 'B' LPCI/CCSW loop was found to be operable and capable of providing core cooling. Cause of the event was due to personnel error. Operator personnel was instructed in the prudent policy of performing required surveillances before taking safety equipment out-of-service.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1) Dresden Nuclear Power Station	DOCKET NUMBER (2) 0 5 0 0 0 2 4 9 8 4 - 0 1 4 - 0 1 0	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 4	0 1 4	0 1 0	0 2	OF 0	2

TEXT (If more space is required, use additional NRC Form 308A's) (17)

During normal operation, LPCI/CCSW heat exchanger delta P valve M0 3-1501-3A was taken out-of-service for limit switch adjustments, rendering the 'A' LPCI/CCSW loop inoperable. No ECCS surveillances were completed prior to rendering the 'A' loop inoperable, violating Technical Specification 4.5.B.3. DOS 1500-3 (Containment Cooling Pump Test) was immediately initiated on the 'B' LPCI/CCSW loop. The '3A' valve limits were adjusted and the valve was returned to service. The total time the valve was out-of-service was an hour and a half. Safety significance was minimal since the LPCI/CCSW 'B' loop was found to be operable and capable of providing core cooling. Cause of the event was personnel error. The Operating personnel was instructed in the prudent policy of performing required surveillances before taking safety equipment out-of-service. First reported occurrence of this type.



Commonwealth Edison

Dresden Nuclear Power Station

R.R. #1

Morris, Illinois 60450

Telephone 815/942-2920

October 4, 1984

DJS Ltr # 84-1111

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Licensee Event Report #84-014-00, Docket # 050249, is being submitted as required by Technical Specification 6.6, NUREG 1022 and 10 CFR 50.73(a)(2)(i)(B).

D.J. Scott
Station Superintendent
Dresden Nuclear Power Station

DJS/aw

Enclosure

cc: J.G. Keppler, Regional Administrator, Region III
File/NRC
File/Numerical

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