

LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

JUNE 1984

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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## I. INTRODUCTION

The LaSalle Nuclear Power Station is a Two Unit Facility Located in Marseilles, Illinois. Each Unit is a Boiling Water Reactor with a designed electrical output of 1078 MWe net. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. Unit One is subject to License Number NPF-11, issued on April 17, 1982. The date of initial criticality was June 21, 1982. Unit Two is subject to license number NPF-18, issued on December 16, 1983. The date of initial criticality was March 10, 1984.

This report was compiled by Randy S. Dus telephone number (815)357-6761, extension 324.

## II. MONTHLY REPORT FOR UNIT ONE

### A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

June 1 The Unit started the reporting period shutdown due to a loss of condenser vacuum and a subsequent reactor scram.

June 2-24 The reactor went critical at 1125 hours on June 2. At 0100 hours on June 3, the Main Generator was synchronized to the grid. At 0645 hours on June 3, reactor power was raised to 15%. At 1500 hours on June 3, reactor power was raised to 33%. At 0700 hours on June 4, reactor power was raised to 62%. At 1500 hours on June 5, reactor power was raised to 80%. At 0020 hours on June 9, reactor power was lowered to 27% for the Unit Two "Loss of Offsite Power Test". At 0700 hours on June 9, reactor power was raised to 76%. At 0000 hours on June 11, reactor power was raised to 90%. At 1330 hours on June 11, reactor power was reduced to 48% due to a Reactor recirc pump trip. At 0000 hours on June 12, reactor power was raised to 76%. At 0700 hours on June 13, reactor power was raised to 90%. At 0000 hours on June 14, reactor power was raised to 97%. At 0700 hours on June 23, reactor power was reduced to 74% for a rod sequence change. At 0820 hours on June 24, a reactor scram occurred as a result of a reactor feed pump flow control problem. The reactor was critical for 524 hours and 55 minutes.

June 25-30 The reactor went critical at 0531 hours on June 25. At 1414 hours on June 25, the main generator was synchronized to the grid. At 1500 hours on June 25, reactor power was raised to 44%. At 0700 hours on June 25, reactor power was raised to 63%. At 0000 hours on June 28, reactor power was raised to 90%. The reactor was critical for 138 hours and 29 minutes.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specification.

There were no amendments to the facility license or Technical Specification.

2. Facility or procedure changes requiring NRC approval.

There were no facility or procedure changes requiring NRC approval.

3. Tests and Experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval.

4. Corrective maintenance of safety related equipment.

The following table (Table 1) presents a summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request numbers, LER numbers, Component Name, Cause of Malfunction, Results and Effects on Safe Operation, and Corrective Action.

TABLE 1  
CORRECTIVE MAINTENANCE OF  
SAFETY RELATED EQUIPMENT

LTP-300-7  
Revision 4  
February 29, 1984  
6

| WORK REQUEST | LER | COMPONENT                      | CAUSE OF MALFUNCTION                                  | RESULTS AND EFFECTS<br>ON SAFE OPERATION | CORRECTIVE ACTION             |
|--------------|-----|--------------------------------|---|--|-------------------------------|
| L36832       |     | Drywell temperature recorder.  | Faulty pinion and clutch on drive mechanism.          | None. Redundant channel operable.        | Repaired drive mechanism      |
| L36849       |     | Pipe Tunnel HI A T Isol.       | Instrumentation drift out of calibration.             | RCIC Isolation from DIV II               | Recalibrated Instrumentation. |
| L37213       |     | HPCS water Leg Pump            | Defective bearing                                     | Potential loss of line pressure.         | Replaced pump.                |
| L37340       |     | Drywell ventilation damper     | Damper failed CLOSED.                                 | No effect on safe operation.             | Wired damper OPEN.            |
| L37905       |     | SBGT WRGM                      | Faulty plug connector causing high resistance ground. | Causes intermittent spiking.             | Repaired grounding Problem.   |
| L38085       |     | RCIC Steam Supply Isol. valve. | Valve Packing Leak.                                   | No effect on operation. ALARA concern.   | Repaired leak.                |

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, occurring during the reporting period, June 1 through June 30, 1984. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

| <u>Licensee Event Report Number</u> | <u>Date</u> | <u>Title of Occurrence</u>                                  |
|-------------------------------------|-------------|---|
| 84-025-00                           | 5/17/84     | Lack of Positive Control on Entry into high radiation area. |
| 84-026-00                           | 5/17/84     | Electrical Cable Penetrations Inoperable                    |
| 84-027-00                           | 5/20/84     | Missed Off Gas Hydrogen Sample                              |
| 84-028-00                           | 5/13/84     | RCIC Isolation Inboard System                               |
| 84-029-00                           | 5/31/84     | Rx Scram from low vacuum trip of turbine generator.         |
| 84-030-00                           | 5/31/84     | Rx Water Cleanup high differential flow isolation.          |

#### D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions



1. OPERATING DATA REPORT

DOCKET NO. 050-373  
UNIT LaSalle One  
DATE July 10, 1984  
COMPLETED BY Randy S. Dus  
TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: June 1984 GROSS HOURS IN REPORTING PERIOD: 720
  2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): 3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
  3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
  4. REASONS FOR RESTRICTION (IF ANY):
- |   | THIS MONTH     | YR TO DATE     | CUMULATIVE     |
|---|----------------|----------------|----------------|
| 5. NUMBER OF HOURS REACTOR WAS CRITICAL     | <u>663.4</u>   | <u>3257.8</u>  | <u>3257.8</u>  |
| 6. REACTOR RESERVE SHUTDOWN HOURS           | <u>56.6</u>    | <u>1076.3</u>  | <u>1076.3</u>  |
| 7. HOURS GENERATOR ON LINE                  | <u>641.1</u>   | <u>3097.7</u>  | <u>3097.7</u>  |
| 8. UNIT RESERVE SHUTDOWN HOURS              | <u>0.0</u>     | <u>1.0</u>     | <u>1.0</u>     |
| 9. GROSS THERMAL ENERGY GENERATED (MWH)     | <u>1806185</u> | <u>8574125</u> | <u>8574125</u> |
| 10. GROSS ELEC. ENERGY GENERATED (MWH)      | <u>583251</u>  | <u>2813123</u> | <u>2813123</u> |
| 11. NET ELEC. ENERGY GENERATED (MWH)        | <u>557739</u>  | <u>2677044</u> | <u>2677044</u> |
| 12. REACTOR SERVICE FACTOR                  | <u>92.1%</u>   | <u>74.6%</u>   | <u>74.6%</u>   |
| 13. REACTOR AVAILABILITY FACTOR             | <u>100%</u>    | <u>99.2%</u>   | <u>99.2%</u>   |
| 14. UNIT SERVICE FACTOR                     | <u>89.0%</u>   | <u>70.9%</u>   | <u>70.9%</u>   |
| 15. UNIT AVAILABILITY FACTOR                | <u>89.0%</u>   | <u>70.9%</u>   | <u>70.9%</u>   |
| 16. UNIT CAPACITY FACTOR (USING MDC)        | <u>74.8%</u>   | <u>59.2%</u>   | <u>59.2%</u>   |
| 17. UNIT CAPACITY FACTOR (USING DESIGN MWe) | <u>71.9%</u>   | <u>56.9%</u>   | <u>56.9%</u>   |
| 18. UNIT FORCED OUTAGE RATE                 | <u>11.0%</u>   | <u>24.5%</u>   | <u>24.5%</u>   |
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)  
On October 1, 1984 there will be a four week outage to inspect the drywell and perform scheduled surveillances.
  20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: Same
  21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):
- |                      | FORECAST        | ACHIEVED       |
|----------------------|-----------------|----------------|
| INITIAL CRITICALITY  | <u>        </u> | <u>6/21/82</u> |
| INITIAL ELECTRICITY  | <u>        </u> | <u>9/04/82</u> |
| COMMERCIAL OPERATION | <u>        </u> | <u>1/1/84</u>  |

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-373  
UNIT: LASALLE ONE  
DATE: JULY 10, 1984  
COMPLETED BY: Randy S. Dus  
TELEPHONE: (815) 357-6761

MONTH: June 1984

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

|     |      |
|-----|------|
| 1.  | 0    |
| 2.  | 0    |
| 3.  | 197  |
| 4.  | 623  |
| 5.  | 783  |
| 6.  | 916  |
| 7.  | 918  |
| 8.  | 799  |
| 9.  | 650  |
| 10. | 846  |
| 11. | 729  |
| 12. | 864  |
| 13. | 1028 |
| 14. | 1049 |
| 15. | 1060 |
| 16. | 1061 |

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

|     |      |
|-----|------|
| 17. | 1027 |
| 18. | 1044 |
| 19. | 1046 |
| 20. | 1009 |
| 21. | 1013 |
| 22. | 1002 |
| 23. | 712  |
| 24. | 281  |
| 25. | 127  |
| 26. | 657  |
| 27. | 786  |
| 28. | 960  |
| 29. | 1044 |
| 30. | 1041 |
| 31. |      |

INSTRUCTIONS

On this form list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line.) In such cases the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

LTP-300-7  
Revision 3  
March 1, 1983  
9 (Final)

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH JUNE 1984

DOCKET NO. 050-373  
UNIT NAME LaSalle One  
DATE July 10, 1984  
COMPLETED BY Randy S. Dus  
TELEPHONE (815)357-6761

| NO. | DATE   | TYPE                      | DURATION<br>(HOURS) | REASON (1) | METHOD OF<br>SHUTTING DOWN<br>THE REACTOR OR<br>REDUCING POWER | CORRECTIVE<br>ACTIONS/COMMENTS   |
|-----|--------|---------------------------|---------------------|------------|--|--|
|     |        | F: FORCED<br>S: SCHEDULED |                     |            |  |  |
| 13  | 840531 | F                         | 49.0                | A          | 4  | Continuation of outage from previous month. Blown loop seals resulting in loss of condenser vacuum.  |
| 14  | 840624 | F                         | 29.9                | A          | 3  | Rx scram on low water level resulting from the loss of a Rx feed pump. Problem stemmed from a malfunction in the Rx water level control logic. Work performed under work request L38185. |

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Revision 3  
March 1, 1983  
9 (Final)

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH MAY 1984

DOCKET NO. 050-373  
UNIT NAME LaSalle One  
DATE July 10, 1984  
COMPLETED BY Randy S. Dus  
TELEPHONE (815)357-6761

| NO.  | DATE   | TYPE<br>F: FORCED<br>S: SCHEDULED | DURATION<br>(HOURS) | REASON (1) | METHOD OF<br>SHUTTING DOWN<br>THE REACTOR OR<br>REDUCING POWER | CORRECTIVE<br>ACTIONS/COMMENTS   |
|------|--------|-----------------------------------|---------------------|------------|--|--|
| 11*  | 840506 | S                                 | 0.0                 | H          | 5  | Power reduction for a rod sequence change.   |
| 12*  | 840527 | F                                 | 0.0                 | A          | 5  | Power reduction due to a TDRFP trip.   |
| 13** | 840531 | F                                 | 7.7                 | A          | 3  | Loss of condenser vacuum as a result of blown SJAE & S.P.E. Loop Seals resulted in Turbine trip & Reactor scram. Procedure revisions in progress to ensure loop seals remain filled. |

\* Added July 10, previously omitted.

\*\* Was outage No. 9.

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Revision 3  
March 1, 1983  
9 (Final)

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH APRIL 1984

DOCKET NO. 050-373  
UNIT NAME LaSalle One  
DATE July 10, 1984  
COMPLETED BY Randy S. Dus  
TELEPHONE (815)357-6761

| NO.  | DATE   | TYPE                      | DURATION<br>(HOURS) | REASON | METHOD OF<br>SHUTTING DOWN<br>THE REACTOR OR<br>REDUCING POWER | CORRECTIVE<br>ACTIONS/COMMENTS  |
|------|--------|---------------------------|---------------------|--------|--|---|
|      |        | F: FORCED<br>S: SCHEDULED |                     |        |  |   |
| 9*   | 840407 | S                         | 0.0                 | H      | 5  | Power reduction for rod sequence change.                                  |
| 10** | 840414 | F                         | 34.3                | G(1)   | 3  | Procedures were changed to more clearly define the operational setpoints. |

(1) The operator changed the setpoint of the reactor water level control below the operational level allowed by existing procedures for paralleling an additional feedwater pump. This resulted in a low reactor water level scram.

\* Added July 10, previously omitted.

\*\* Was outage No. 8.

LTP-300-7  
Revision 3  
March 1, 1983  
9 (Final)

ATTACHMENT E

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH MARCH 1984

DOCKET NO. 050-373  
UNIT NAME LaSalle One  
DATE July 10, 1984  
COMPLETED BY Randy S. Dus  
TELEPHONE (815)357-6761

| NO. | DATE   | TYPE                      | DURATION<br>(HOURS) | REASON (1) | METHOD OF<br>SHUTTING DOWN<br>THE REACTOR OR<br>REDUCING POWER | CORRECTIVE<br>ACTIONS/COMMENTS  |
|-----|--------|---------------------------|---------------------|------------|--|---|
|     |        | F: FORCED<br>S: SCHEDULED |                     |            |  |   |
| 6   | 840213 | S                         | 156.5               | B          | 4  | Condenser Boot Seal<br>and extraction steam<br>expansion joint repaired.  |
| 7   | 840308 | F                         | 286.7               | F          | 1  | Temporary drywell vent-<br>ilation ductwork evaluated<br>for loading on containment<br>structural members.<br>Analysis O.K. No<br>corrective action taken.<br>Remained shutdown to<br>perform electrical cable<br>butt splices inspection<br>required by NRC. |
| 8*  | 840328 | F                         | 0.0                 | A          | 5  | Power reduction as a<br>result of feedwater<br>heater string<br>isolations.   |

\* Added July 10, 1984, previously omitted.

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief valve operations for Unit One.

There were no relief valve operations for Unit One for this reporting period.

## 2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

| <u>OUTAGE NO.</u> | <u>EQUIPMENT</u>                            | <u>PURPOSE OF OUTAGE</u>           |
|-------------------|---|------------------------------------|
| 1-491-84          | HPCS Water Leg Pump                         | Install new pump                   |
| 1-509-84          | LPCS Water Leg Pump                         | General Maintenance                |
| 1-518-84          | RHR Water Leg Pump<br>Min Flow Bypass Stop  | Repair Valve Motor                 |
| 1-519-84          | RHR Water Leg Pump<br>Min Flow Bypass Stop  | Repair Valve Motor                 |
| 1-537-84          | RHR Water Leg Pump<br>Min Flow Bypass Stop  | Repair Valve Motor                 |
| 1-538-84          | RHR Water Leg Pump<br>Min Flow Bypass Stop  | Repair Valve Motor                 |
| 1-542-84          | HPCS D/G Petter<br>Diesel Air<br>Compressor | General Maintenance                |
| 1-545-84          | LPCS Water Leg Pump                         | Lubricate Coupling                 |
| 1-558-84          | RHR Service Water<br>Pump                   | Replace Outboard Bearing<br>Gasket |

## 3. Off-Site Dose Calculation Manual

There were no changes to the off-site dose calculations manual during this reporting period.

## 4. Radioactive Waste Treatment Systems.

There were no changes to the Radioactive Waste Treatment System during this reporting period.



LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

JUNE 1984

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

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    - 1. Safety/Relief Valve Operations
    - 2. ECCS System Outages
    - 3. Off-Site Dose Calculation Manual Changes
    - 4. Major Changes to Radioactive Waste Treatment System

## I. INTRODUCTION

The LaSalle Nuclear Power Station is a Two Unit Facility Located in Marseilles, Illinois. Each Unit is a Boiling Water Reactor with a designed electrical output of 1078 MWe net. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. Unit One is subject to License Number NPF-11, issued on April 17, 1982. The unit commenced commercial generation of power on January 1, 1984. Unit Two is subject to license number NPF-18, issued on December 16, 1983. The date of initial criticality was March 10, 1984.

This report was compiled by Randy S. Dus, telephone number (815)357-6761, extension 324.

II. MONTHLY REPORT FOR UNIT TWO

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT TWO

June 1-6 The unit started the reporting period at 25% power with the main generator synchronized to the grid. At 0330 hours on June 1, reactor power was raised to 45%. At 1307 hours on June 5, reactor power was reduced to 24% because of feedwater chemistry problems. At 0530 hours on June 6, a reactor scram occurred while performing an instrument maintenance surveillance. The reactor was critical for 125 hours and 30 minutes.

June 6-10 At 2150 hours on June 6, the reactor was critical. At 0515 hours on June 7, the main generator was synchronized to the grid. At 1400 hours on June 7, reactor power was raised to 24%. At 0700 hours on June 8, reactor power was raised to 44%. At 1700 hours on June 8, reactor power was reduced to 24% due to feedwater chemistry problems. At 2325 hours on June 8, a reactor scram occurred as a result of the "Loss of Offsite Power" test. The reactor was critical for 49 hours and 35 minutes.

June 11-16 At 1100 hours on June 11, the reactor was critical. At 0400 hours on June 12, the main generator was synchronized to the grid. At 0700 hours on June 12, reactor power was raised to 39%. At 2300 hours on June 12, reactor power was raised to 50%. At 1500 hours on June 15, reactor power was reduced to 12% for maintenance work. At 2000 hours on June 15, the main generator was removed from the grid. At 0200 hours on June 16, the reactor was shutdown for maintenance work on the condensate booster system. The reactor was critical for 111 hours and 0 minutes.

June 17-30 At 0825 hours on June 17, the reactor was critical. At 1025 hours on June 18, the main generator was synchronized to the grid. At 1500 hours on June 18, reactor power was raised to 18%. At 1500 hours on June 19, reactor power was raised to 47%. At 1500 hours on June 22, reactor power was raised to 62%. At 0700 hours on June 24, reactor power was reduced to 54% for flux shaping. At 2300 hours on June 26, reactor power was raised to 70%. At 1900 hours on June 28, reactor power was reduced to 45% for bypass valve testing. At 0000 hours on June 29, reactor power was reduced to 22% due to feedwater chemistry problems. At 2030 hours on June 30, reactor power was raised to 64%. The reactor was critical for 327 hours and 35 minutes.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specifications.

There were no amendments to the facility license or Technical Specifications during the reporting period.

2. Facility or procedure changes requiring NRC approval.

There were no facility or procedure changes requiring NRC approval during the reporting period.

3. Tests and experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval during the reporting period.

4. Corrective Maintenance of Safety Related Equipment.

The following table (Table 1) presents a summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request numbers, LER Numbers, Component Name, cause of malfunction, results and effects on safe operation, and corrective action.

TABLE 1  
CORRECTIVE MAINTENANCE OF  
SAFETY RELATED EQUIPMENT

LTP-300-7  
Revision 4  
February 29, 1984  
6

| WORK REQUEST | LER | COMPONENT                          | CAUSE OF MALFUNCTION   | RESULTS AND EFFECTS<br>ON SAFE OPERATION     | CORRECTIVE ACTION                  |
|--------------|-----|------------------------------------|--|--|------------------------------------|
| L37604       |     | Rx Water<br>Sample Valve           | Valve does not isolate<br>as required/defective<br>Solenoid          | Potential loss of Contain-<br>ment integrity | Repaired solenoids                 |
| L37625       |     | Div II Post<br>Loca O <sub>2</sub> | Instrumentation Drift<br>causing false oxygen<br>readings in drywell | None. Redundant channel<br>still available.  | Recalibrated<br>instrument.        |
| L37626       |     | APRM Flow<br>Comparators           | Power supply was tripped<br>resulting in false<br>reading.           | Causes rod block                             | Restored power to<br>Instrument.   |
| L37865       |     | HPCS Disch<br>Relief Valve         | Bellows Assembly within<br>valve body failed.                        | Potential loss of containment<br>integrity.  | Replaced failed<br>bellows.        |
| L37937       |     | D/G Immersion<br>Heater            | Defective Transformer<br>heater cannot energize                      | Potential D/G Damage upon<br>sudden start.   | Replaced defective<br>transformer. |

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit Two, occurring during the reporting period, June 1 through June 30, 1984. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

| <u>Licensee Event Report Number</u> | <u>Date</u> | <u>Title of Occurrence</u>                              |
|-------------------------------------|-------------|---|
| 84-018-00                           | 5/17/84     | U2 HPCS Pump Breaker Malfunction                        |
| 84-019-00                           | 5/9/84      | Missed four hour hydrogen sampling of Off Gas system.   |
| 84-020-00                           | 5/21/84     | U2 Generator lockout and reactor scram.                 |
| 84-021-00                           | 5/15/84     | Reactor Water Cleanup high differential flow isolation. |
| 84-022-00                           | 5/21/84     | Loss of positive control on High Rad. Gate.             |
| 84-023-00                           | 5/29/84     | Reactor water cleanup isolations.                       |
| 84-024-00                           | 5/31/84     | Loss of RCIC Control and Instrument Power.              |
| 84-025-00                           | 6/6/84      | Rx Scram due to personnel error.                        |

#### D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions



1. OPERATING DATA REPORT

DOCKET NO. 050-374  
UNIT LaSalle Two  
DATE July 10, 1984  
COMPLETED BY Randy S. Dus  
TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: June 1984 GROSS HOURS IN REPORTING PERIOD: 720  
2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): 3323 MAX DEPEND CAPACITY  
(MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078  
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A  
4. REASONS FOR RESTRICTION (IF ANY):  
5. NUMBER OF HOURS REACTOR WAS CRITICAL 613.7 THIS MONTH YR TO DATE 1910.5 CUMULATIVE 1910.5  
6. REACTOR RESERVE SHUTDOWN HOURS 106.3 792.3 792.3  
7. HOURS GENERATOR ON LINE 557.3 1138.4 1138.4  
8. UNIT RESERVE SHUTDOWN HOURS 0.0 0.0 0.0  
9. GROSS THERMAL ENERGY GENERATED (MWH) 892493 1488958 1488958  
10. GROSS ELEC. ENERGY GENERATED (MWH) 249632 376516 376516  
11. NET ELEC. ENERGY GENERATED (MWH) 235000 348093 348093  
12. REACTOR SERVICE FACTOR N/A N/A N/A  
13. REACTOR AVAILABILITY FACTOR N/A N/A N/A  
14. UNIT SERVICE FACTOR N/A N/A N/A  
15. UNIT AVAILABILITY FACTOR N/A N/A N/A  
16. UNIT CAPACITY FACTOR (USING MDC) N/A N/A N/A  
17. UNIT CAPACITY FACTOR (USING DESIGN  
MWe) N/A N/A N/A  
18. UNIT FORCED OUTAGE RATE N/A N/A N/A  
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)  
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: N/A  
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):  
INITIAL CRITICALITY FORECAST ACHIEVED  
INITIAL ELECTRICITY 3/10/84  
COMMERCIAL OPERATION 4/20/84  
Aug. 84

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-374  
UNIT: LASALLE TWO  
DATE: July 10, 1984  
COMPLETED BY: Randy S. Dus  
TELEPHONE: (815) 357-6761  
MONTH: June 1984

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

1. 401  
2. 427  
3. 419  
4. 413  
5. 268  
6. 41  
7. 164  
8. 345  
9. 0  
10. 0  
11. 0  
12. 273  
13. 436  
14. 463  
15. 334  
16. 0

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

17. 0  
18. 60  
19. 268  
20. 415  
21. 448  
22. 510  
23. 535  
24. 522  
25. 513  
26. 610  
27. 704  
28. 650  
29. 180  
30. 394  
31. \_\_\_\_\_

INSTRUCTIONS

On this form list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line.) In such cases the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

LTP-300-7  
Revision 3  
March 1, 1983  
9 (Final)

ATTACHMENT E  
3. UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH JUNE 1984

DOCKET NO. 050-374  
UNIT NAME LaSalle Two  
DATE July 10, 1984  
COMPLETED BY Randy S. Dus  
TELEPHONE (815)357-6761

| NO. | DATE   | TYPE                      | DURATION<br>(HOURS) | REASON (1) | METHOD OF<br>SHUTTING DOWN<br>THE REACTOR OR<br>REDUCING POWER | CORRECTIVE<br>ACTIONS/COMMENTS   |
|-----|--------|---------------------------|---------------------|------------|--|--|
|     |        | F: FORCED<br>S: SCHEDULED |                     |            |  |  |
| 16  | 840605 | F                         | 0.0                 | H          | 5  | Power Reduction as a<br>result of Rx Water<br>High Conductivity  |
| 17  | 840606 | F                         | 23.8                | G          | 3  | Personnel error while<br>working on Rx water<br>level instrumentation.<br>System upset resulted in<br>turbine trip & Rx scram<br>(Re: LER No. 84-025-00) |

LTP-300-7  
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ATTACHMENT E

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH JUNE 1984

DOCKET NO. 050-374  
UNIT NAME LaSalle Two  
DATE July 10, 1984  
COMPLETED BY Randy S. Dus  
TELEPHONE (815)357-6761

| NO. | DATE   | TYPE                      | DURATION<br>(HOURS) | REASON (1) | METHOD OF<br>SHUTTING DOWN<br>THE REACTOR OR<br>REDUCING POWER | CORRECTIVE<br>ACTIONS/COMMENTS  |
|-----|--------|---------------------------|---------------------|------------|--|---|
|     |        | F: FORCED<br>S: SCHEDULED |                     |            |  |   |
| 18  | 840608 | S                         | 76.6                | B          | 3  | Unit Shutdown as a result of the "Loss of Offsite Power" Test. (STP-31-2) |
| 19  | 840615 | S                         | 62.4                | B          | 1  | Unit Shutdown to perform maintenance work.                                |
| 20  | 840629 | F                         | 0.0                 | H          | 5  | Power reduction as a result of Rx water high conductivity.                |

## E. UNIQUE REPORTING REQUIREMENTS

### 1. Safety/Relief Valve Operations for Unit Two.

| <u>DATE</u> | <u>VALVES<br/>ACTUATED</u> | <u>NO &amp; TYPE<br/>ACTUATIONS</u> | <u>PLANT<br/>CONDITION</u> | <u>DESCRIPTION<br/>OF EVENT</u> |
|-------------|----------------------------|-------------------------------------|----------------------------|---------------------------------|
| 6/8/84      | 2B21-F013S                 | 27 AUTO                             | 1140 psig                  | Rx Scram for STP-31-2           |
| 6/13/84     | 2B21-F013A                 | 1 MANUAL                            | 960 psig                   | STP-26-2                        |
| 6/13/84     | 2B21-F013B                 | 1 MANUAL                            | 960 psig                   | STP-26-2                        |
| 6/13/84     | 2B21-F013F                 | 1 MANUAL                            | 960 psig                   | STP-26-2                        |
| 6/13/84     | 2B21-F013G                 | 1 MANUAL                            | 960 psig                   | STP-26-2                        |
| 6/13/84     | 2B21-F013H                 | 1 MANUAL                            | 960 psig                   | STP-26-2                        |
| 6/13/84     | 2B21-F013J                 | 1 MANUAL                            | 960 psig                   | STP-26-2                        |
| 6/13/84     | 2B21-F013K                 | 1 MANUAL                            | 960 psig                   | STP-26-2                        |
| 6/13/84     | 2B21-F013L                 | 1 MANUAL                            | 960 psig                   | STP-26-2                        |
| 6/13/84     | 2B21-F013M                 | 1 MANUAL                            | 960 psig                   | STP-26-2                        |
| 6/15/84     | 2B21-F013N                 | 1 MANUAL                            | 960 psig                   | STP-26-2                        |
| 6/15/84     | 2B21-F013P                 | 1 MANUAL                            | 960 psig                   | STP-26-2                        |
| 6/15/84     | 2B21-F013C                 | 1 MANUAL                            | 960 psig                   | STP-26-2                        |
| 6/15/84     | 2B21-F013D                 | 1 MANUAL                            | 960 psig                   | STP-26-2                        |
| 6/15/84     | 2B21-F013E                 | 1 MANUAL                            | 960 psig                   | STP-26-2                        |
| 6/15/84     | 2B21-F013R                 | 1 MANUAL                            | 960 psig                   | STP-26-2                        |
| 6/15/84     | 2B21-F013S                 | 1 MANUAL                            | 960 psig                   | STP-26-2                        |
| 6/15/84     | 2B21-F013U                 | 1 MANUAL                            | 960 psig                   | STP-26-2                        |
| 6/15/84     | 2B21-F013V                 | 1 MANUAL                            | 960 psig                   | STP-26-2                        |

## 2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

| <u>OUTAGE NO.</u> | <u>EQUIPMENT</u>                             | <u>PURPOSE OF OUTAGE</u> |
|-------------------|--|--------------------------|
| 2-729-84          | 2A RHR Pump                                  | General Maintenance      |
| 2-731-84          | RHR HX Shell Side<br>Vent Downstream<br>Stop | Adjust Limit Switch      |
| 2-762-84          | HPCS Discharge<br>Line Relief Valve          | Repair Ruptured Bellows  |
| 2-777-84          | RHR Water Leg<br>Pump Check Valve            | General Maintenance      |
| 2-789-84          | 2C RHR Pump                                  | Sample Oil               |
| 2-799-84          | LPCS Pump                                    | Lubrication              |
| 2-800-84          | RHR Pump                                     | Lubrication              |

## 3. Off-Site Dose Calculation Manual

There were no changes to the off-site dose calculations manual during this reporting period.

## 4. Radioactive Waste Treatment Systems.

There were no changes to the Radioactive Waste Treatment System during this reporting period.



**Commonwealth Edison**  
LaSalle County Nuclear Station  
Rural Route #1, Box 220  
Marseilles, Illinois 61341  
Telephone 815/357-6761

July 6, 1984

Director, Office of Management Information  
and Program Control  
United States Nuclear Regulatory Commission  
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering  
LaSalle County Nuclear Power Station for the period covering June 1 through  
June 30, 1984.

Very truly yours,

G. J. Diederich  
Superintendent  
LaSalle County Station

GJD/RSD/crh

Enclosure

xc: J. G. Keppler, NRC, Region III  
NRC Resident Inspector LaSalle  
Gary Wright, Ill. Dept. of Nuclear Safety  
D. P. Galie, CEC  
D. L. Farrar, CEC  
INPO Records Center  
Ron A. Johnson, PIP Coordinator SNED  
W. R. Jackson, GE Resident

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