



Duquesne Light

Nuclear Construction Division
Robinson Plaza, Building 2, Suite 210
Pittsburgh, PA 15205

2NRC-4-163

(412) 787-5141

(412) 923-1960

Teletype (412) 787-2629

October 10, 1984

United States Nuclear Regulatory Commission
Washington, DC 20555

ATTENTION: Mr. George W. Knighton, Chief
Licensing Branch 3
Office of Nuclear Reactor Regulation

SUBJECT: Beaver Valley Power Station - Unit No. 2
Docket No. 50-412
Supplemental Submittal in Response to Draft SER Open Items 78
and 81

Gentlemen:

In letter 2NRC-4-042, dated April 12, 1984, DLC provided responses to the NRC Chemical Technology Section's Draft SER Open Items 78 through 81, including the appropriate FSAR revisions. In a telephone conference with the Chemical Technology Section reviewer, Mr. J. Wing, on August 21, 1984, Mr. Wing requested that DLC provide additional information in response to Open Items 78 and 81. Specifically, the additional information requested is as follows:

Open Item 78: Describe the modifications that will be made to the generic Westinghouse Owners Group Post-Accident Core Damage Assessment Methodology (dated February 1984) to make it the BVPS-2 plant specific core damage estimate procedure.

Open Item 81: Provide appropriate documentation to verify that the PASS instrumentation is capable of performing its intended function in an accident environment.

The supplemental response to Open Item 78 is provided in Attachment 1 and the supplemental response to Open Item 81 is provided in Attachment 2. Based on the aforementioned August 21, 1984, telephone conference with the NRC, it is DLC's understanding that, with this submittal, Open Item 78 becomes a confirmatory item and Open Item 81 is closed, unless further notification indicating otherwise is received from the NRC.

DUQUESNE LIGHT COMPANY

SUBSCRIBED AND SWORN TO BEFORE ME THIS
10th DAY OF October, 1984.

Anita Elaine Reiter
Notary Public

ANITA ELAINE REITER, NOTARY PUBLIC
ROBINSON TOWNSHIP, ALLEGHENY COUNTY

JDO/wjs MY COMMISSION EXPIRES OCTOBER 20, 1986
Attachments

By E. J. Woolever
E. J. Woolever
Vice President

cc: Ms. M. Ley, Project Manager (w/a)
Mr. E. A. Licitra, Project Manager (w/o)
Mr. G. Walton, NRC Resident Inspector (w/o)

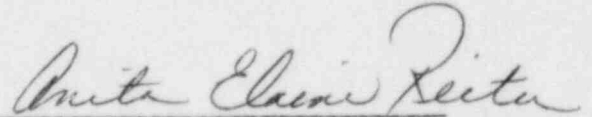
CHG. LPDR
FOR
NSIC
NTIS } NOW PROP
ONLY

8410160490 841010
PDR ADDCK 05000412
E PDR

Boo1
1/1

COMMONWEALTH OF PENNSYLVANIA)
) SS:
COUNTY OF ALLEGHENY)

On this 10th day of October, 1984, before me, a
Notary Public in and for said Commonwealth and County, personally appeared
E. J. Woolever, who being duly sworn, deposed and said that (1) he is Vice
President of Duquesne Light, (2) he is duly authorized to execute and file
the foregoing Submittal on behalf of said Company, and (3) the statements
set forth in the Submittal are true and correct to the best of his
knowledge.


Notary Public

ANITA ELAINE REITER, NOTARY PUBLIC
ROBINSON TOWNSHIP, ALLEGHENY COUNTY
MY COMMISSION EXPIRES OCTOBER 20, 1986

ATTACHMENT 1

Supplemental Response to Draft SER Open Item 78 (Section 9.3.2.2) - Procedure to estimate extent of core damage:

DLC is planning to make the following modifications to the generic Westinghouse Owners Group Post Accident Core Damage Assessment Methodology (dated February 1984) to make it the BVPS-2 plant specific core damage estimate procedure:

- (1) Delete references to 2-loop, 4-loop, and ice condenser containment plants in Tables 2-2, 2-3, 2-3-1, 2-8, 4-1, and in Figure 3-1.
- (2) Ratio source inventories given in Tables 2-2, 2-3, 2-3-1, and 2-8 from the 2900 Mwt standard 3-loop plant to the 2652 Mwt core of BVPS-2.
- (3) Replace Figure 3-2 with a drawing of BVPS-2 core thermocouple locations.
- (4) Delete references to 2-loop, 4-loop, and ice condenser containment plants in Sections 3.1 and 5.0.

The above modifications are minor and could be done now. The only remaining modification required is to Section 3.3, "Containment Radiation Monitors and Core Damage." This section will be finalized when more details are available on the BVPS-2 containment radiation monitor and its location within containment. It is anticipated that the final version of the BVPS-2 post-accident core damage estimate procedure will be available for submittal to the NRC by June 1985.

ATTACHMENT 2

Supplemental Response to Draft SER Open Item 81 (Section 9.3.2.2) - Performance of PASS instrumentation and analytical procedures:

Table OI81-1 (attached) lists the BVPS-2 PASS instrumentation test reports prepared by NUS Corporation, the associated instruments, and the analysis performed by each of the instruments. The NUS test reports, which are attached, verify that the BVPS-2 PASS instrumentation is capable of performing its intended function in an accident environment.

TABLE OI81-1

<u>Test Report</u>	<u>Instrument</u>	<u>Analysis Performed by Instrument</u>
Report No. NSAC/46, dated April 1982: An Evaluation of On-Line Boron Analyzers	Ionics DigiChem 3000	Reactor Coolant Boron
Report No. R-27-4-1-1, dated April 1981: Development of Procedures and Analysis Methods for Post-Accident Reactor Coolant Samples	Dionex Ion Chromatograph 2010i	Reactor Coolant Chloride
Report No. R-27-10-0-2, dated October 1980: Analyses for Dissolved Hydrogen, Dissolved Oxygen, Boron, Chloride, pH, and Conductivity under Normal and Post-Accident Conditions		Reactor Coolant Dissolved Hydrogen; Containment Atmospheric Oxygen
Report No. R-27-3-1-1A, dated May 1981: An Evaluation of the Baseline System for Determination of Total Gas Concentration During Normal or Post-Accident Conditions	Baseline Gas Chromatograph	
Report No. CSD-84-144, revised March 26, 1984: Testing of Dissolved Oxygen Analyzers for Post-Accident Analysis Application	Orbisphere Dissolved Oxygen Analyzer 2606	Reactor Coolant Dissolved Oxygen
Report No. CSD-2-84-13, dated February 7, 1984: Testing of L&N 7075-3 pH Receiver and Electrode Preamplifier and Meridian pH Electrode for PASS Applications	L&N pH Monitor 7075-3	Reactor Coolant pH