

AVERAGE DAILY UNIT POWER LEVEL

Completed by L. K. Miller

Docket No. 50-311
 Unit Name Salem # 2
 Date July 10, 1984
 Telephone 609-935-6000
 Extension 4455

Month June 1984

Day Average Daily Power Level
 (MWe-NET)

1	<u>0</u>
2	<u>0</u>
3	<u>0</u>
4	<u>0</u>
5	<u>0</u>
6	<u>0</u>
7	<u>0</u>
8	<u>0</u>
9	<u>0</u>
10	<u>0</u>
11	<u>0</u>
12	<u>0</u>
13	<u>0</u>
14	<u>574</u>
15	<u>1073</u>
16	<u>1097</u>

Day Average Daily Power Level
 (MWe-NET)

17	<u>1070</u>
18	<u>1108</u>
19	<u>1102</u>
20	<u>1086</u>
21	<u>1087</u>
22	<u>1094</u>
23	<u>1077</u>
24	<u>1087</u>
25	<u>1095</u>
26	<u>1100</u>
27	<u>1107</u>
28	<u>1045</u>
29	<u>1103</u>
30	<u>1100</u>
31	<u> </u>

Pg. 8,1-7 R1

8407250310 840630
 PDR ADOCK 05000311
 R PDR

OPERATING DATA REPORT

Docket No. 50-311
 Date July 10, 1984
 Telephone 935-6000
 Extension 4455

Completed by L. K. Miller

Operating Status

1. Unit Name	<u>Salem No. 2</u>	<u>Notes</u>
2. Reporting Period	<u>June 1984</u>	
3. Licensed Thermal Power (MWt)	<u>3411</u>	
4. Nameplate Rating (Gross MWe)	<u>1162</u>	
5. Design Electrical Rating (Net MWe)	<u>1115</u>	
6. Maximum Dependable Capacity (Gross MWe)	<u>1149</u>	
7. Maximum Dependable Capacity (Net MWe)	<u>1106</u>	
8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason <u>N/A</u>		
9. Power Level to Which Restricted, if any (Net MWe) <u>N/A</u>		
10. Reasons for Restrictions, if any <u>N/A</u>		
	<u>This Month</u>	<u>Year to Date</u>
		<u>Cumulative</u>
11. Hours in Reporting Period	<u>720</u>	<u>4367</u>
12. No. of Hrs. Reactor was Critical	<u>421.0</u>	<u>1918.3</u>
13. Reactor Reserve Shutdown Hrs.	<u>0</u>	<u>1442.9</u>
14. Hours Generator On-Line	<u>405.4</u>	<u>1769.6</u>
15. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>
16. Gross Thermal Energy Generated (MWH)	<u>1345678</u>	<u>5743709</u>
17. Gross Elec. Energy Generated (MWH)	<u>449880</u>	<u>1904810</u>
18. Net Elec. Energy Generated (MWH)	<u>425465</u>	<u>1784158</u>
19. Unit Service Factor	<u>56.3</u>	<u>40.5</u>
20. Unit Availability Factor	<u>56.3</u>	<u>40.5</u>
21. Unit Capacity Factor (using MDC Net)	<u>53.4</u>	<u>36.9</u>
22. Unit Capacity Factor (using DER Net)	<u>53.0</u>	<u>36.6</u>
23. Unit Forced Outage Rate	<u>43.7</u>	<u>59.5</u>
24. Shutdowns scheduled over next 6 months (type, date and duration of each) <u>N/A</u>		
25. If shutdown at end of Report Period, Estimated Date of Startup: <u>N/A</u>		
26. Units in Test Status (Prior to Commercial Operation):		
Initial Criticality	<u>Forecast</u> <u>6/30/80</u>	<u>Achieved</u> <u>8/2/80</u>
Initial Electricity	<u>9/1/80</u>	<u>6/3/81</u>
Commercial Operation	<u>9/24/80</u>	<u>10/13/81</u>

8-1-7.R2

UNIT SHUTDOWN AND POWER REDUCTIONS
REPORT MONTH June 1984

Docket No. 50-311
Unit Name Salem No.2
Date July 10, 1984
Telephone 609-935-6000
Extension 4455

Completed by L.K. Miller

No.	Date	Type 1	Duration Hours	Reason 2	Method of Shutting Down Reactor	License Event Report	System Code 4	Component Code 5	Cause and Corrective Action to Prevent Recurrence
84-170	05-30	F	314.6	A	1	-	SF	VALVEX	High Pressure Safety Injection Core Injection/Core Spray
84-172	06-14	F	4.7	A	5	-	HH	INSTRU	Speed Control, Feedwater Pump
84-190	06-23	F	.5	B	5	-	HF	FILTER	Traveling Screen/ Trash Rack/ Canal Screen
84-192	06-23	F	.2	B	5	-	HF	FILTER	Traveling Screen/ Trash Rack Canal Screen

1
F: Forced
S: Scheduled

2 Reason
A-Equipment Failure-explain
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & Licensing Exam
F-Administrative
G-Operational Error-explain
H-Other-explain

3 Method
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuation of
Previous Outage
5-Load Reduction
9-Other

4 Exhibit G
Instructions
for Prepara-
tion of Data
Entry Sheets
for Licensee
Event Report
(LER) File
(NUREG 0161)

5 Exhibit 1
Salem as
Source

MAJOR PLANT MODIFICATIONS
REPORT MONTH JUNE 1984

DOCKET NO: 50-311
UNIT NAME: SALEM 2
DATE: JULY 10, 1984
COMPLETED BY: L. K. MILLER
TELEPHONE: (609) 339-4455

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
2ET-0944	Circulating Water	Provide electrical power feed to vendor supplied heat transfer research unit.
2EC-1008	Main Transformers	Replace vacuum switch on COPS tank with new model switch and arrangement. Also relocate vacuum switch alarm.
2EC-1387	SEC	Install noise suppression.
2EC-1620A	Reactor Coolant System-Wide Range RTD Recorders	Replace existing recorders with seismically qualified recorders. Instrument Nos. TA-0014, TA-0016, TA-0036 and TA-0038.
2EC-1724	Chemical & Volume Control	Replace the 2CV45 and 2CV50 valves.

* DESIGN CHANGE REQUEST
8-1-7.R1

MAJOR PLANT MODIFICATIONS
REPORT MONTH JUNE 1984

DOCKET NO.: 50-311
UNIT NAME: Salem 2
DATE: July 10, 1984
COMPLETED BY: L.K. Miller
TELEPHONE: 609/339-4455

DCR NO.	10CFR 50.59	SAFETY EVALUATION
2ET-0944	This test requires no change to existing systems, nor does it affect any presently performed safety analysis. No unreviewed safety or environmental questions are involved.	
2EC-1008	This change in the alarms improves plant reliability by decreasing operator response time. No unreviewed safety or environmental questions are involved.	
2EC-1387	Implementation of this DCR does not involve any unreviewed safety or environmental questions.	
2EC-1620A	All potential realistic failures have been considered and are not applicable. No unreviewed safety or environmental questions are involved.	
2EC-1724	Addition of the blind flange will eliminate two potential paths of unidentified leakage from the reactor Coolant System. No unreviewed safety or environmental questions are involved.	

* Design Change Request

PSE&G SALEM GENERATING STATION
SAFETY RELATED WORK ORDER LOG

SALEM UNIT NO. 2

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION	
954241	OD	2	NO. 24 OPAT	
			FAILURE DESCRIPTION:	INDICATOR STUCK IN MID SCALE (050484)
			CORRECTIVE ACTION:	REPLACED SEVERAL CAPACITORS AND RECALIBRATED (050484)
954117	OD	2	NO. 24 STEAM GENERATOR LEVEL INDICATION	
			FAILURE DESCRIPTION:	CHANNEL III IS READING 3% LOWER THAN THE OTHER CHANNELS (050784)
			CORRECTIVE ACTION:	REPAIRED ROOT VALVE PACKING LEAK AND PERFORMED RECALIBRATION (052284)
900108-0	MD	2	VALVE 22WG10	
			FAILURE DESCRIPTION:	BLOCKAGE IN SUCTION TO WASTE GAS COMER (062584)
			CORRECTIVE ACTION:	REPLACED DIAPHRAGM (070384)
9900019-9				
	MD	2	4KV VITAL BUS	
			FAILURE DESCRIPTION:	REMOVE 83 RELAY FROM 1A 4KV VITAL BUS AND INSTALL ON 2C 4KV VITAL BUS (051584)
			CORRECTIVE ACTION:	REMOVED RELAY FROM 1A AND INSTALLED IN 2C, INSTALLED REBUILT RELAY IN 1A AND TESTED (060784)

UNIT 2

99022966 MD 2 2C SAFEGUARDS EQUIPMENT CONTROL CABINET

FAILURE DESCRIPTION: SPURIOUS ALARMS (061184)

CORRECTIVE ACTION: INCREASED DC VOLTAGE IAW MD84-3215 (061284)

84-05-29-550-0

CD 2 NO. 21 MOTOR GENERATOR SET

FAILURE DESCRIPTION: HIGH VIBRATION (052948)

CORRECTIVE ACTION: REPLACED INBOARD AND OUTBOARD BEARINGS (060584)

SALEM UNIT NO. 2
OPERATIONS SUMMARY REPORT
JUNE 1984

Unit No. 2 began the month shutdown in order to replace 21SJ16 and 22SJ16 valves. These valves, which are in the ECCS flow path, were considered inoperable after radiographs indicated the valve plug and stem orientation were not proper. Inspection of the valves after their removal revealed that the weld that holds the disk nut to the disk was missing. The Unit remained in Mode 5 until June 5, 1984 when plant heatup commenced. No load pressure and temperature of 2235 psig and 547°F was reached on June 6, 1984. A plant cooldown to approximately 367°F was reached on June 6, 1984. A plant cooldown to approximately 367°F was initiated on June 7, 1984 to facilitate changeout of No. 23 CVCU motor. No. 23 CFCU motor failed to start in slow speed during surveillance testing thereby requiring its repair or replacement. No. 23 CFCU was subsequently replaced with No. 14 CFCU, however it too failed to start in slow speed. No. 22 CFCU has been placed in the No. 23 CFCU location. No. 12 CFCU has been placed in the No. 22 CFCU location. The apparent cause of the CFCU failures to start in slow speed was misalignment of the rotor to stator air gap. The plant recommenced heatup on June 13, 1984. Criticality was achieved at 1059 hours on June 13, 1984. The Unit was synchronized on June 14, 1984 at 0238 hours and reached full power on June 15, 1984. The Unit remained at full power until June 23, 1984 when the Unit load was decreased to 64% at 0932 hours due to heavy grass loading on the Circulating Water Traveling Screens. The Unit returned to full power at 2110 hours on June 23, 1984 where it remained for the remainder of the period.

REFUELING INFORMATION

COMPLETED BY: L.K. MillerDOCKET NO.: 50-311UNIT NAME: Salem 2DATE: July 10, 1984TELEPHONE: 609/935-6000EXTENSION: 4455Month June 1984

1. Refueling information has changed from last month:
YES _____ NO X
2. Scheduled date for next refueling: January 5, 1985
3. Scheduled date for restart following refueling: March 17, 1985
4. A) Will Technical Specification changes or other license amendments be required?
YES _____ NO _____
NOT DETERMINED TO DATE 6/1/84
- B) Has the reload fuel design been reviewed by the Station Operating Review Committee?
YES _____ NO X
If no, when is it scheduled? November 1984
5. Scheduled date(s) for submitting proposed licensing action:
December 1984 (if required)
6. Important licensing considerations associated with refueling:
NONE
7. Number of Fuel Assemblies:
A) Incore 193
B) In Spent Fuel Storage 72
8. Present licensed spent fuel storage capacity: 1170
Future spent fuel storage capacity: 1170
9. Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: March 2004

8-1-7.R4



Public Service Electric and Gas Company P.O. Box E Hancocks Bridge, New Jersey 08038

Salem Generating Station

July 10, 1984

Director, Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
SALEM NO. 2
DOCKET NO. 50-311

In Compliance with Section 6.9, Reporting Requirements for the Salem Technical Specifications, 10 copies of the following monthly operating reports for the month of June 1984 are being sent to you.

Average Daily Unit Power Level
Operating Data Report
Unit Shutdowns and Power Reductions
Major Plant Modification
Safety Related Work Orders
Operating Summary
Refueling Information

Sincerely yours,

J. M. Zupko, Jr.
General Manager - Salem Operations

LKM:sbh

cc: Dr. Thomas E. Murley
Regional Administrator USNRC
Region I
631 Park Avenue
King of Prussia, PA 19406

Director, Office of Management
Information and Program Control
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Enclosures
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IE24
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The Energy People