

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)  
Callaway Plant Unit 1

DOCKET NUMBER (2)

0 5 0 0 0 4 8 3 1 OF 0 2

PAGE (3)

TITLE (4)  
Inadvertent Engineered Safety Feature Actuation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)									
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)							
0	9	0	7	8	4	8	4	0	3	5	0	0	0	0	0	0		
0	9	0	7	8	4	8	4	0	3	5	0	0	1	0	0	6	8	4

OPERATING MODE (9)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																																																						
3	<table border="1"><thead><tr><th>20.402(b)</th><th>20.406(a)(1)(i)</th><th>20.406(a)(1)(ii)</th><th>20.406(a)(1)(iii)</th><th>20.406(a)(1)(iv)</th><th>20.406(a)(1)(v)</th><th>20.406(c)</th><th>50.36(c)(1)</th><th>50.36(c)(2)</th><th>50.73(a)(2)(i)</th><th>50.73(a)(2)(ii)</th><th>50.73(a)(2)(iii)</th><th>50.73(a)(2)(iv)</th><th>50.73(a)(2)(v)</th><th>50.73(a)(2)(vi)</th><th>50.73(a)(2)(vii)</th><th>50.73(a)(2)(viii)(A)</th><th>50.73(a)(2)(viii)(B)</th><th>50.73(a)(2)(ix)</th><th>73.71(b)</th><th>73.71(c)</th><th>OTHER (Specify in Abstract below and in Text, NRC Form 366A)</th></tr></thead><tbody><tr><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td>X</td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td></tr></tbody></table>										20.402(b)	20.406(a)(1)(i)	20.406(a)(1)(ii)	20.406(a)(1)(iii)	20.406(a)(1)(iv)	20.406(a)(1)(v)	20.406(c)	50.36(c)(1)	50.36(c)(2)	50.73(a)(2)(i)	50.73(a)(2)(ii)	50.73(a)(2)(iii)	50.73(a)(2)(iv)	50.73(a)(2)(v)	50.73(a)(2)(vi)	50.73(a)(2)(vii)	50.73(a)(2)(viii)(A)	50.73(a)(2)(viii)(B)	50.73(a)(2)(ix)	73.71(b)	73.71(c)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)										X													
20.402(b)	20.406(a)(1)(i)	20.406(a)(1)(ii)	20.406(a)(1)(iii)	20.406(a)(1)(iv)	20.406(a)(1)(v)	20.406(c)	50.36(c)(1)	50.36(c)(2)	50.73(a)(2)(i)	50.73(a)(2)(ii)	50.73(a)(2)(iii)	50.73(a)(2)(iv)	50.73(a)(2)(v)	50.73(a)(2)(vi)	50.73(a)(2)(vii)	50.73(a)(2)(viii)(A)	50.73(a)(2)(viii)(B)	50.73(a)(2)(ix)	73.71(b)	73.71(c)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)																																		
									X																																														

LICENSEE CONTACT FOR THIS LER (12)  
NAME  
Charles D. Naslund - Superintendent, I&C

TELEPHONE NUMBER

AREA CODE

3 1 4 6 7 6 - 8 5 0 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs	CAUSE	SYSTEM
X	J B	L I I	D 2 3 2	N							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 9/7/84 a Feedwater Isolation Actuation occurred while the plant was in Mode 3. The actuation of this Engineered Safety Feature (ESF) resulted from high level signals received from two of four steam generator (S/G) "B" level indicators. Upon receipt of the Feedwater Isolation Signal (FWIS), the required ESF equipment functioned properly.

One of the two level indicators had failed high due to instrument valve manifold leakage. The second level indicator spiked high during testing of a main steam flow loop sharing the same sensing line.

Testing was discontinued on the main steam flow loop upon initiation of the FWIS. Personnel involved discussed the tie between the common tap instruments and testing resumed with no further incidents. The valve manifold was replaced and the pertinent procedures were revised to include caution statements relative to the common tap instrument tie. This incident is considered an isolated case which requires no further corrective action.

There was no damage to plant equipment or release of radioactivity as a result of this incident. At no time did this event pose a threat to the public health or safety.

8410160331 841006  
PDR ADOCK 05000483  
S PDRIE22  
1/1

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (6)

PAGE (3)

Callaway Plant Unit 1

0 5 0 0 0 4 8 3 8 4 - 0 3 5 - 0 0 0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 365A's) (17)

At 1030 CDT on 9/7/84, a Feedwater Isolation Actuation occurred while the plant was in Mode 3. The actuation of this Engineered Safety Feature (ESF) resulted from high level signals received from two of four steam generator (S/G) "B" level indicators. Upon receipt of the Feedwater Isolation Signal (FWIS), the required ESF equipment functioned properly.

Level indicator AE-LI-552 had failed in the high condition on 9/6/84 due to leakage in the instrument valve manifold. The cause of the leakage is unknown and the instrument valve manifold (Part No. 12067N) was manufactured by Dragon Valve Inc. At 1030 CDT on 9/7/84, level indicator AE-LI-529 spiked high during testing on main steam flow loop AB-F-522. Valving operations performed on the flow loop inadvertently caused the level transmitter to spike because the flow loop and the indicator share the same sensing line. With two high S/G level signals present, the two-out-of-four logic necessary to initiate a FWIS was satisfied and the feedwater isolation occurred.

Testing was discontinued on the main steam flow loop upon initiation of the FWIS. After the personnel involved discussed the tie between the common tap instruments, testing was resumed with no further incidents. Normal feedwater flow was restored at 1055 on 9/7/84.

Level indicator AE-LI-522 functioned properly after the instrument valve manifold was replaced on 9/9/84. The pertinent I&C flow loop calibration procedures have been revised to include caution statements indicating the common tap instrument ties which led to the spiking of AE-LI-529. This incident is considered an isolated case which requires no further corrective action.

There was no damage to plant equipment or release of radioactivity as a result of this incident. At no time did this event pose a threat to the public health or safety.

Previous occurrences: none

UNION ELECTRIC COMPANY  
CALLAWAY PLANT

MAILING ADDRESS:  
P.O. BOX 620  
FULTON, MO. 65251

October 6, 1984

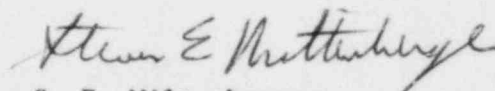
U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

ULNRC-939

DOCKET NUMBER 50-483  
CALLAWAY PLANT UNIT 1  
FACILITY OPERATING LICENSE NPF-25  
LICENSEE EVENT REPORT 84-035-00  
INADVERTENT ENGINEERED SAFETY FEATURES ACTUATION

Gentlemen:

The enclosed Licensee Event Report is submitted pursuant  
to 10 CFR 50.73(a)(2)(iv) concerning an inadvertent Engineered Safety  
Features Actuation.

  
S. E. Miltenberger  
Manager, Callaway Plant

CDN/WRR/JWK/drs  
Enclosure

cc: Distribution attached

IE22  
11

cc distribution for ULNRC-939

Mr. James G. Keppler  
Regional Administrator  
Office of Inspection & Enforcement  
U.S. Nuclear Regulatory Commission  
Region III  
799 Roosevelt Road  
Glen Ellyn, IL 60137

American Nuclear Insurers  
c/o Dottie Sherman, Library  
The Exchange Suite 245  
270 Farmington Avenue  
Farmington, CT 06032

Records Center  
Institute of Nuclear Power Operations  
Suite 1500  
1100 Circle 75 Parkway  
Atlanta, GA 30339

NRC Resident Inspector  
Missouri Public Service Commission  
D. F. Schnell  
J. F. McLaughlin  
J. E. Davis (Z40LER)  
D. W. Capone  
F. D. Field  
R. L. Powers  
A. C. Passwater/D. E. Shafer/D. J. Walker  
G. A. Hughes  
W. R. Robinson (QA Record)  
C. D. Naslund  
D. C. Poole  
R. A. McAleenan  
L. K. Robertson (470) (NSRB)  
Merlin Williams, Wolf Creek  
SEM Chrono  
3456-0021.6  
3456-0260  
Z40ULNRC  
G56.37  
N. Date