

OPERATING DATA REPORT

DOCKET NO. 50-315
 DATE 9-5-84
 COMPLETED BY W.T. Gillet
 TELEPHONE 616-465-590

OPERATING STATUS

1. Unit Name: Donald C. Cook 1
2. Reporting Period: August 1984
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1152
5. Design Electrical Rating (Net MWe): 1030
6. Maximum Dependable Capacity (Gross MWe): 1056
7. Maximum Dependable Capacity (Net MWe): 1020
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe):
10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744	5855	84,743
12. Number Of Hours Reactor Was Critical	492.3	5146.8	62,764.8
13. Reactor Reserve Shutdown Hours	0	0	463
14. Hours Generator On-Line	466.2	5088.8	61,432.5
15. Unit Reserve Shutdown Hours	0	0	321
16. Gross Thermal Energy Generated (MWH)	1,349,657	15,524,904	181,218,617
17. Gross Electrical Energy Generated (MWH)	426,470	5,070,850	58,997,140
18. Net Electrical Energy Generated (MWH)	409,192	4,882,754	56,763,130
19. Unit Service Factor	62.7	86.9	74.3
20. Unit Availability Factor	62.7	86.9	74.3
21. Unit Capacity Factor (Using MDC Net)	53.9	81.8	67.4
22. Unit Capacity Factor (Using DER Net)	53.4	81.0	64.6
23. Unit Forced Outage Rate	5.2	7.6	7.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

Ten Year ISI Surveillance and Refueling Outage Schedule to Start on March 1, 1985.

Estimated duration Four Months.

25. If Shut Down At End Of Report Period, Estimated Date of Restart:

26. Units In Test Status (Prior to Commercial Operation):

Forecast

Achieved

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

8410160197 840831
 PDR ADDOCK 05000315
 R PDR

IE 24
 111

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-315

UNIT 1

DATE 9/4/84

COMPLETED BY A. Might

TELEPHONE (616) 465-5901

MONTH August 1984

DAY AVERAGE DAILY POWER LEVEL
(MWE-Net)

1	-
2	-
3	-
4	-
5	-
6	-
7	-
8	-
9	-
10	-
11	239
12	1007
13	1021
14	604
15	53
16	677

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17	923
18	978
19	1017
20	1021
21	1014
22	1011
23	877
24	548
25	546
26	659
27	988
28	1007
29	991
30	985
31	865

INSTRUCTIONS

On this format list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH AUGUST, 1984

DOCKET NO. 50-315
 UNIT NAME D.C. Cook - Unit 1
 DATE 9-13-84
 COMPLETED BY B.A. Svensson
 TELEPHONE 616/465-5901
 PAGE 1 of 3

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
225 Cont'd.	840727	S	250.6	B	1	N.A.	ZZ	ZZZZZZ	The Unit was removed from service to perform required ice condenser surveillance and to repair leaking pressurizer safety valves. The pressurizer safety valves were replaced with rebuilt valves. The Unit was returned to service on 840811 and reached 100% power on 840812. Total length of the outage was 347.6 hours. A reactor trip with Train "A" safety injection occurred from 100% power due to a failure of vital A.C. Instrument Bus, CRID IV Inverter. The inverter failure was due to water vapor laden air being blown into the inverter cabinet by a temporary ventilation blower. The water source was due to a leak. The inverter was replaced with a spare. The Unit was
226	840814	F	27.2	A	3	84-018-00	EF	INSTRU	

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

⁵
 Exhibit I - Same Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

INSTRUCTIONS

This report should describe all plant shutdowns during the report period. In addition, it should be the source of explanation of significant dips in average power levels. Each significant reduction in power level (greater than 20% reduction in average daily power level for the preceding 24 hours) should be noted, even though the unit may not have been shut down completely¹. For such reductions in power level, the duration should be listed as zero, the method of reduction should be listed as 4 (Other), and the Cause and Corrective Action to Prevent Recurrence column should explain. The Cause and Corrective Action to Prevent Recurrence column should be used to provide any needed explanation to fully describe the circumstances of the outage or power reduction.

NUMBER. This column should indicate the sequential number assigned to each shutdown or significant reduction in power for that calendar year. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported. Until a unit has achieved its first power generation, no number should be assigned to each entry.

DATE. This column should indicate the date of the start of each shutdown or significant power reduction. Report as year, month, and day. August 14, 1977 would be reported as 770814. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported.

TYPE. Use "F" or "S" to indicate either "Forced" or "Scheduled," respectively, for each shutdown or significant power reduction. Forced shutdowns include those required to be initiated by no later than the weekend following discovery of an off-normal condition. It is recognized that some judgment is required in categorizing shutdowns in this way. In general, a forced shutdown is one that would not have been completed in the absence of the condition for which corrective action was taken.

DURATION. Self-explanatory. When a shutdown extends beyond the end of a report period, count only the time to the end of the report period and pick up the ensuing down time in the following report periods. Report duration of outages rounded to the nearest tenth of an hour to facilitate summation. The sum of the total outage hours plus the hours the generator was on line should equal the gross hours in the reporting period.

REASON. Categorize by letter designation in accordance with the table appearing on the report form. If category H must be used, supply brief comments.

METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER. Categorize by number designation

¹Note that this differs from the Edison Electric Institute (EEI) definitions of "Forced Partial Outage" and "Scheduled Partial Outage." For these terms, EEI uses a change of 30 MW as the break point. For larger power reactors, 30 MW is too small a change to warrant explanation.

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LICENSEE EVENT REPORT =. Reference the applicable reportable occurrence pertaining to the outage or power reduction. Enter the first four parts (event year, sequential report number, occurrence code and report type) of the five part designation as described in Item 17 of Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161). This information may not be immediately evident for all such shutdowns, of course, since further investigation may be required to ascertain whether or not a reportable occurrence was involved.) If the outage or power reduction will not result in a reportable occurrence, the positive indication of this lack of correlation should be noted as not applicable (N/A).

SYSTEM CODE. The system in which the outage or power reduction originated should be noted by the two digit code of Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161).

Systems that do not fit any existing code should be designated XX. The code ZZ should be used for those events where a system is not applicable.

COMPONENT CODE. Select the most appropriate component from Exhibit I - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161), using the following criteria:

- A. If a component failed, use the component directly involved.
- B. If not a component failure, use the related component: e.g., wrong valve operated through error; list valve as component.
- C. If a chain of failures occurs, the first component to malfunction should be listed. The sequence of events, including the other components which fail, should be described under the Cause and Corrective Action to Prevent Recurrence column.

Components that do not fit any existing code should be designated XXXXXX. The code ZZZZZZ should be used for events where a component designation is not applicable.

CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE. Use the column in a narrative fashion to amplify or explain the circumstances of the shutdown or power reduction. The column should include the specific cause for each shutdown or significant power reduction and the immediate and contemplated long term corrective action taken, if appropriate. This column should also be used for a description of the major safety-related corrective maintenance performed during the outage or power reduction including an identification of the critical path activity and a report of any single release of radioactivity or single radiation exposure specifically associated with the outage which accounts for more than 10 percent of the allowable annual values.

For long textual reports continue narrative on separate paper and reference the shutdown or power reduction for this narrative.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH AUGUST, 1984

DOCKET NO. 50-315
 UNIT NAME D.C. Cook - Unit 1
 DATE 9-13-84
 COMPLETED BY B.A. Svensson
 TELEPHONE 616/465-5901
 PAGE 2 of 3

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
226 Cont'd. 227	840817	F	0	H	4	N.A.	HH	TURBIN	returned to service on 840815 and re-actor power reached 100% on 840816. Reactor power was reduced from 100% to 80% for approximately 18 hours to reduce the loading of the east main feed pump turbine due to a high vibration problem on the turbine in-board bearing.
228	840823	F&S	0	B	4	N.A.	HH	HTEXCH & TURBIN	Reactor power was reduced from 100% to 58 to first remove the west main feed pump turbine to repair condenser tube leak. One tube was plugged. The west MFPT was then placed in service and the east MFPT removed from service to repair the turbine inboard bearing oil leak and the cause for the high vibration still being experienced on this bearing. The east MFPT was returned to service and

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 B Maintenance or Test
 C Refueling
 D Regulatory Restriction
 E Operator Training & License Examination
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 G Operational Error (Explain)
 H Other (Explain)

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 COMPLETED BY B.A. Svensson
 TELEPHONE 616/465-5901
 PAGE 3 of 3

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228 Cont'd 229	840831	F	0	S	4	N.A.	HH	TURBIN	reactor power increased to 99% on 840826. Reactor power was reduced to 57% to remove the east main feed pump turbine from service to make alignment checks and replace the turbine in-board bearing with a new design (elliptical) bearing. The east MFPT was returned to service on 840902 and reactor power returned to 100% on 840904. The oil leak has been repaired and no vibration excursions have been experienced since the bearing replacement.

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Docket No.: 50-315
Unit Name: D.C. Cook Unit 1
Completed By: G. J. Peak
Telephone: (616) 465-5901
Date: 09/07/84
Page: 1 of 2

MONTHLY OPERATING ACTIVITIES - AUGUST, 1984

HIGHLIGHTS:

The unit entered the reporting period in Mode 5 with the Reactor Coolant System at the half loop elevation due to an ice condenser surveillance outage. The surveillance requirements were completed and the Unit was returned to service and loaded to 100% power. The Crid IV inverter failed on 8-14-84 causing a reactor trip and Train A safety injection. The inverter was replaced and the unit was subsequently returned to 100% power where it was operating when the reporting period ended. Major power reductions occurred to remove the West Main Feed Pump from service for a condenser tube leak check and to remove the East Main Feed Pump from service due to high vibration.

Total electrical generation for the month was 426,470 MWH.

SUMMARY:

- 8-06-84 The Reactor Coolant System was filled and vented at 2035 hours following an ice condenser surveillance outage.
- 8-07-84 Mode 4 was entered at 1818 hours.
- 8-08-84 Mode 3 was entered at 0623 hours.
- 8-10-84 The reactor was critical at 1450 hours.
- 8-11-84 The main turbine was rolled at 0856 hours. The main generator was paralleled at 1035 hours.
- 8-12-84 Power reached 100% at 0600 hours.
- 8-14-84 The unit tripped and a Train A safety injection occurred at 1529 hours due to the failure of crid IV.
- 8-15-84 Crid IV was declared operable at 0944 hours and the reactor was made critical at 1225 hours. Mode 1 was entered at 1445 hours. The main turbine was rolled at 1734 hours. The main generator was paralleled at 1841 hours.
- 8-16-84 Power reached 100% at 2133 hours.
- 8-17-84 Power was reduced to 80% at 1350 hours due to high vibration on the East Main Feed Pump.

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- 8-18-84 Power was returned to 100% at 0825 hours.
- 8-23-84 Power was reduced to 58% at 1905 hours to remove the West Main Feed Pump from service for a condenser tube leak check.
- 8-24-84 The West Main Feed Pump was returned to service at 0005 hours and the East Main Feed Pump was removed from service at 0629 hours to repair a bearing oil seal leak.
- 8-26-84 The East Main Feed Pump was returned to service at 1224 hours and a power increase began at 1410 hours. Power reached 99% at 2249 hours when the East Main Feed Pump indicated high vibration. Therefore, power was reduced to 88% at 2257 hours.
- 8-27-84 The unit was loaded to 100% at 0456 hours. Power was reduced to 92% at 1657 hours due to high vibration on the East Main Feed Pump. The unit was returned to 100% power at 2146 hours.
- 8-29-84 Power was reduced to 92% at 0610 hours due to high vibration on the East Main Feed Pump. Power was returned to 100% at 1027 hours.
- 8-30-84 Power was reduced to 97% at 0710 hours due to high vibration on the East Main Feed Pump.
- 8-31-84 Power was further reduced to 57% at 2025 hours in order to remove the East Main Feed Pump from service.

The Control Room Cable Vault Halon System remains inoperable as of 1400 hours on 4-5-83. The backup CO₂ System for the Control Room Cable Vault remains operable.

DOCKET NO.	50 - 315
UNIT NAME	D. C. Cook - Unit No. 1
DATE	9-13-84
COMPLETED BY	B. A. Svensson
TELEPHONE	(616) 465-5901
PAGE	1 of 1

MAJOR SAFETY-RELATED MAINTENANCE

AUGUST, 1984

- M-1 Checked operation of 1 "E" ESW Pp Breaker. Found defective anti-pump relay. Replaced relay and performed electrical test to verify proper operation.
- M-2 Repaired Pressurizer PORV's, NRV-151, NRV-152 and NRV-153, as required to eliminate seat leakage. Valves performed satisfactorily when retested.
- M-3 Replaced plunger, throat bushing, and repacked reciprocal charging pump. Ran pump to verify packing leakage within acceptable limits.
- M-4 Investigated reason for breaker tripping when attempting to operate emergency boration valve, QMO-410. Found open winding in limit torque motor and replaced motor. Retest was performed and valve functioned properly.
- M-5 Repaired ice condenser intermediate deck doors to pass lifting test. Door 11-C was replaced. Surveillance test was performed after repairs to verify operability.
- C&I-1 The critical control room power inverter failed. Capacitor C2 and fuse FU2 were replaced. The diodes and SCR's were tested and were OK. The inverter was restarted and returned to service.
- C&I-2 Steam dump URV-120 did not open in cooldown mode. The solenoid valve's coil was open. The coil was replaced. The solenoid valve then worked properly.
- C&I-3 Containment isolation valve ECR-32 would not close and ECR-31 would not indicate it was closed. The solenoid valves for both valves were leaking by and were rebuilt. The valves then worked properly and ECR-31 and ECR-32 were tested for proper closing times.
- C&I-4 QRV-51, auxiliary spray to the pressurizer had failed closed. The coil of solenoid valve XS0-51 was measured and found open. The coil was replaced and the valve was verified to open and close properly.
- C&I-5 Loop 4, Tavg indication failed low. Dynamic compensator module TY-442A was found to be defective and was replaced and calibrated. The ΔT /Tavg system was verified to be operable.



INDIANA & MICHIGAN ELECTRIC COMPANY

Donald C. Cook Nuclear Plant
P.O. Box 458, Bridgman, Michigan 49106

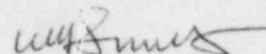
September 13, 1984

Director, Office Of Management Information
and Program Control
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Gentlemen:

Pursuant to the requirements of Donald C. Cook Nuclear Plant Unit 1
Technical Specification 6.9.1.6, the attached Monthly Operating
Report for the Month of August, 1984 is submitted.

Sincerely,


W. G. Smith, Jr.
Plant Manager

WGS:ab

Attachments

cc: J. E. Dolan
M. P. Alexich
R. W. Jurgensen
NRC Region III
E. R. Swanson
R. O. Bruggee (NSAC)
R. C. Callen
S. J. Mierzwa
R. F. Kroeger
B. H. Bennett
J. D. Huebner
J. H. Hennigan
Z. Cordero
R. F. Hering
J. F. Stietzel
PNSRC File
INPO Records Center
ANI Nuclear Engineering Department

IE24
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