

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-346  
 UNIT Davis-Besse #1  
 DATE 9/4/84  
 COMPLETED BY Bilal M. Sarsour  
 TELEPHONE (419)259-5000  
ext. 384

MONTH AUGUST, 1984

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	804
2	804
3	798
4	804
5	804
6	799
7	797
8	799
9	801
10	797
11	800
12	802
13	801
14	799
15	800
16	796

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	794
18	797
19	795
20	703
21	769
22	767
23	761
24	742
25	721
26	722
27	698
28	645
29	673
30	673
31	610

## INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

8410160181 840831  
 PDR ADDCK 05000346  
 R PDR

# OPERATING DATA REPORT

DOCKET NO. 50-346  
 DATE 9/4/84  
 COMPLETED BY Bilal Sarsour  
 TELEPHONE (419) 259-5000  
 Ext. 384

## OPERATING STATUS

1. Unit Name: Davis-Besse #1
2. Reporting Period: August, 1984
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 915
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 918
7. Maximum Dependable Capacity (Net MWe): 874
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe):
10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744	5,855.0	53,376.0
12. Number Of Hours Reactor Was Critical	744	5,276.4	32,778.9
13. Reactor Reserve Shutdown Hours	0.0	134.8	4,014.1
14. Hours Generator On-Line	744	5,236.9	31,388.7
15. Unit Reserve Shutdown Hours	0.0	0.0	1,732.5
16. Gross Thermal Energy Generated (MWH)	1,862,784	13,426,681	74,470,495
17. Gross Electrical Energy Generated (MWH)	601,231	4,390,990	24,683,183
18. Net Electrical Energy Generated (MWH)	568,838	4,143,374	23,142,073
19. Unit Service Factor	100	89.4	58.8
20. Unit Availability Factor	100	89.4	62.1
21. Unit Capacity Factor (Using MDC Net)	87.5	81.0	49.6
22. Unit Capacity Factor (Using DER Net)	84.4	78.1	47.9
23. Unit Forced Outage Rate	0.0	10.6	17.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

Refueling outage: Scheduled start 9/14/84, scheduled end 12/26/84

25. If Shut Down At End Of Report Period, Estimated Date of Startup:
26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY  
 INITIAL ELECTRICITY  
 COMMERCIAL OPERATION

Forecast

Achieved

## UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH AUGUST, 1984

DOCKET NO. 50-346  
 UNIT NAME Davis-Besse #1  
 DATE 9/4/84  
 COMPLETED BY Bilal Sarsour  
 TELEPHONE (419)259-5000  
 ext. 384

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
NO UNIT SHUTDOWNS OR SIGNIFICANT POWER REDUCTIONS									

<sup>1</sup>  
 F: Forced  
 S: Scheduled

<sup>2</sup>  
 Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance or Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Continuation from Previous Month  
 5-Load Reduction  
 9-Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions  
 for Preparation of Data  
 Entry Sheets for Licensee  
 Event Report (LER) File (NUREG-  
 0161)

<sup>5</sup>  
 Exhibit I - Same Source

## OPERATIONAL SUMMARY

August, 1984

Reactor power was maintained at approximately 94% power with the generator gross load at approximately  $840 \pm 10$  MWe (Reactor power was limited to 94% due to an inoperable main steam safety valve) until 0801 hours on August 20, 1984 when a plant runback to approximately 62% power was initiated due to dropped control rod 4-7 (group 4 rod 7) during the control rod drive breaker test. The cause of the event was a blown fuse in phase AA of control rod 4-7. The control rod fuse was replaced and the rod was repulled.

Reactor power was slowly increased and attained approximately 90% power at 1900 hours on August 20, 1984 and maintained at this power level (Reactor power was limited to 90% to begin the coastdown for the end of cycle 4) until 1300 hours on August 24, 1984. Reactor power was then decreased in a step change manner for the coastdown.

Reactor power was being maintained at approximately 78% until 1301 hours on August 31, 1984, when a plant runback to approximately 55% was initiated due to dropped control rod 5-1 caused by a blown fuse.

Reactor power was slowly increased to approximately 75% power and maintained for the rest of the month.

## REFUELING INFORMATION

DATE: August, 1984

1. Name of facility: Davis-Besse Unit 1
2. Scheduled date for next refueling shutdown: September 14, 1984
3. Scheduled date for restart following refueling: December 26 1984
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what in general will these be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

Ans: Expect the Reload Report to require standard reload fuel design Technical Specification changes (3/4.1 Reactivity Control Systems and 3/4.2 Power Distribution Limits).

5. Scheduled date(s) for submitting proposed licensing action and supporting information: July, 1984
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

Ans: None identified to date.

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.

(a) 177 (b) 140 - Spent Fuel Assemblies

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

Present: 735 Increase size by: 0 (zero)

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

Date: 1993 - assuming ability to unload the entire core into the spent fuel pool is maintained.

COMPLETED FACILITY CHANGE REQUEST

FCR NO: 80-045

SYSTEM: Radiation Monitoring System

COMPONENT: Radiation Monitors

CHANGE, TEST, OR EXPERIMENT: A 10 CFR 50.59 review was performed for radiation levels in room 317A, outside containment emergency hatch; Room 400 area outside the containment equipment hatch; Room 600, purge inlet equipment room area; Rooms 601 and 602, main steam line rooms; Room 303, the #3 Mechanical Penetration Room (near the door to annulus) and in the east side of the region inside the containment vessel on level 603'. Maximum limits were established and radiation zone maps were revised per Drawings A-11 through A-19 by March 2, 1983.

REASON FOR CHANGE: A review of radiation levels in the annulus and within the containment vessel has assured that the actual dose rates are not greater than those given in the FSAR. This change upgraded FSAR/USAR values to more realistic values including adequate margins.

SAFETY EVALUATION: The safety function of the radiation zone maps is to provide indications of the expected radiation levels for the various station conditions so that appropriate radiation access controls can be implemented to insure that operating personnel are adequately protected. Although the zone maps only provide the expected radiation levels, they should reflect, as closely as possible, the actual measure conditions in the plant. The revised radiation zone maps have satisfactorily accomplished this goal. Since the zone maps were not used as a basis for the evaluation of equipment qualifications, the revision of these zone maps should not have any impact on the equipment qualifications in the plant. It has been concluded that this change did not present any unreviewed safety questions.





September 7, 1984

Log No. K84-1152  
File: RR 2 (P-6-84-07)

Docket No. 50-346  
License No. NPF-3

Mr. Norman Haller, Director  
Office of Management and Program Analysis  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Dear Mr. Haller:

Monthly Operating Report, August 1984  
Davis-Besse Nuclear Power Station Unit 1

Enclosed are ten copies of the Monthly Operating Report for Davis-Besse Nuclear Power Station Unit 1 for the month of August 1984.

If you have any questions, please feel free to contact Bilal Sarsour at (419) 259-5000, Extension 384.

Yours truly,

*Terry D Munay*  
*for*

Stephen M. Quennoz  
Plant Manager  
Davis-Besse Nuclear Power Station

SMQ/BMS/bec

Enclosures

cc: Mr. James G. Keppler, w/1  
Regional Administrator, Region III

Mr. Richard DeYoung, Director, w/2  
Office of Inspection and Enforcement

Mr. Walt Rogers, w/1  
NRC Resident Inspector

LJK/002

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