

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Brunswick Steam Electric Plant Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 3 2 5				PAGE (3) 1 OF 0 2						
TITLE (4) Inoperability of the A Reactor Core Spray Subsystem on Both Units 1 and 2																				
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)										
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)							
0	9	0	6	8	4	8	4	0	2	3	0	0	1	0	0	5	8	4	Brunswick Unit 2	0 5 0 0 0 3 2 4
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																		
POWER LEVEL (10)		20.402(b)				20.406(c)				50.73(a)(2)(iv)				73.71(b)						
1 0 0		20.406(a)(1)(i)				50.36(c)(1)				X 50.73(a)(2)(v)				73.71(c)						
		20.406(a)(1)(ii)				50.36(c)(2)				X 50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 365A)						
		20.406(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)										
		20.406(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)										
		20.406(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(x)										
LICENSEE CONTACT FOR THIS LER (12)																				
NAME M. J. Pastva, Jr., Regulatory Technician										TELEPHONE NUMBER 9 1 1 9 4 5 7 1 - 9 5 2 1										
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																				
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC											
X	BIM	ISIP	TB 4 5 0	Yes																
SUPPLEMENTAL REPORT EXPECTED (14)																				
X YES (If yes, complete EXPECTED SUBMISSION DATE)										NO		EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR				
														1	2	0	3	8	4	

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On September 6, 1984, it was discovered that the common support imbed plate of Unit 2 A Core Spray subsystem hydraulic snubbers 2-E21-2SS31 and 2SS32 was loose and the plate concrete base was cracked and broken. Subsequent inspections revealed the condition existed on the same corresponding piping support imbed plate of the Unit 1 A Core Spray subsystem. At the time of this event, Unit 1 was at 100 percent power and Unit 2 was in a refuel/maintenance outage. In addition, the B Core Spray subsystem on both units and the Low Pressure Coolant Injection subsystems, the High Pressure Coolant Injection System, and the Reactor Core Isolation Cooling System on Unit 1 were operable.

After preliminary plant Engineering investigation and evaluation of this event, the A Core Spray subsystem on each unit was respectively declared inoperable. The subject hydraulic snubber imbed plate on both units was strengthened with wing plates and wedge anchors.

A walkdown of the loop piping of both Core Spray subsystems on each unit has been conducted to determine if other subsystems' piping supports are affected. No other problems were found that affect system operability.

Following further investigation and evaluation concerning this event, an appropriate supplement to this report will be issued.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	OF	
Brunswick Steam Electric Plant Unit 1	05000325	84	023	00	02	OF 02

TEXT (If more space is required, use additional NRC Form 365A's) (17)

On September 6, 1984, following the performance of an inspection of electrical conduit fastening clamps in the Unit 2 Reactor Building, it was determined that the common support imbed plate of the unit's A Core Spray subsystem hydraulic snubbers 2-E21-2SS31 and 2SS32 was loose and the plate concrete base was broken and cracked to approximately 1/8" in depth. Initial investigations indicate the cause of this event was probably water hammer. Subsequent inspections of the Unit 2 B Core Spray subsystem and the Unit 1 A and B Core Spray subsystems revealed that similar conditions also existed on the same corresponding piping support imbed plate of the Unit 1 A Core Spray subsystem. The B Core Spray subsystem on each unit has a different piping configuration. The involved Unit 1 A Core Spray hydraulic snubbers are 1-E21-2SS31 and 2SS32. The hydraulic snubbers of each unit are located on the subsystem piping upstream of the subsystem primary containment outboard isolation valve, 1(2)-E21-F004A. Each pair of snubbers serves to restrain the subsystem lines, 2-12-300 and 41-10-604, on each unit. At the time of these discoveries, Unit 1 was operating at 100 percent power and Unit 2 was in a refueling/maintenance outage.

Following a preliminary plant Engineering investigation and analysis of these discoveries, a determination was made that operability of the involved hydraulic snubbers during an encountered design seismic event could not be verified. As a result, the A Core Spray subsystem on each unit was declared inoperable. At the time, both Low Pressure Coolant Injection (LPCI) Residual Heat Removal (RHR) subsystems and the B Core Spray subsystem on Unit 1 were operable. In addition, the unit High Pressure Coolant Injection (HPCI) System and Reactor Core Isolation Cooling (RCIC) System were operable. On Unit 2, the B Core Spray subsystem was operable.

The subject hydraulic snubber imbed plate on both units has been strengthened by the addition of wing plates and wedge anchors. The piping of both Core Spray subsystems on each unit has been walked down to determine if other subsystems' supports are affected. No other problems were found that affect system operability.

Following further investigation and evaluation concerning this event, an appropriate supplement to this report will be issued on December 3, 1984.



Carolina Power & Light Company

Brunswick Steam Electric Plant  
P. O. Box 10429  
Southport, NC 28461-0429

October 5, 1984

FILE: B09-13510C  
SERIAL: BSEP/84-2104

NRC Document Control Desk  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555

BRUNSWICK STEAM ELECTRIC PLANT UNIT 1  
DOCKET NO. 50-325  
LICENSE NO. DPR-71  
LICENSEE EVENT REPORT 1-84-23

Gentlemen:

In accordance with Title 10 to the Code of Federal Regulations, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-1022, September 1983.

Very truly yours,

C. R. Dietz, General Manager  
Brunswick Steam Electric Plant

MJP/dgr/LETDR1

Enclosure

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