

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Brunswick Steam Electric Plant Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 3 2 1 5 1					PAGE (3) 1 OF 0 1		
TITLE (4) Group 3 Primary Containment Isolation Due to Erroneous Reactor Water Cleanup System Area High Temperature Signal																	
EVENT DATE (6)			LER NUMBER (8)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (9)							
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)				
0 9	0 6	8 4	8 4	0 2	2	0 0	1 0	0 4	8 4					0 5 0 0 0			
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)														
1			20.402(b)				20.406(c)				X 60.73(a)(2)(iv)				73.71(b)		
POWER LEVEL (10)			20.405(a)(1)(i)				60.38(c)(1)				60.73(a)(2)(v)				73.71(c)		
0 1 9 1 9			20.405(a)(1)(iii)				60.38(c)(2)				60.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)		
			20.405(a)(1)(iii)				60.73(a)(2)(i)				60.73(a)(2)(viii)(A)						
			20.405(a)(1)(iv)				60.73(a)(2)(ii)				60.73(a)(2)(viii)(B)						
			20.405(a)(1)(v)				60.73(a)(2)(iii)				60.73(a)(2)(x)						
LICENSEE CONTACT FOR THIS LER (12)																	
NAME M. J. Pastva, Jr., Regulatory Technician										TELEPHONE NUMBER							
										AREA CODE 9 1 1 9 4 5 7 1 - 9 5 2 1							
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																	
CAUSE	SYSTEM	COMPONENT	MANUFAC- TURER	REPORTABLE TO NRCDS		CAUSE	SYSTEM	COMPONENT	MANUFAC- TURER	REPORTABLE TO NRCDS							
X	BID I	MIOD	SIO 15 1 4	No													
SUPPLEMENTAL REPORT EXPECTED (14)																	
YES (If yes, complete EXPECTED SUBMISSION DATE)										X NO			EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 9-6-84, at 1240, the Unit 1 Reactor Water Cleanup (RWCU) System inlet primary containment inboard and outboard isolation valves, 1-G31-F001 and F004, automatically closed due to a Group 3 primary containment isolation resulting from a false indication of an RWCU area high temperature condition. This event was revealed to the Unit 1 Control Operator through the RWCU System "Steam Leak Detection Ambient Temperature High" alarm annunciator on the Unit 1 Control Room Reactor Turbine Gauge Board. At the time, Unit 1 was operating at 99% power.

The erroneous RWCU System area high temperature input originated from RWCU area temperature high isolation instrument, 1-G31-TS-N600F. Individual component breakdown in the N600F instrument module, SCAM Instrument Corp. Part No. 86PTGF-0084, resulted in a failure of the N600F compensation circuitry and temperature converter module, thereby causing the instrument to see an erroneous high temperature condition. The module was replaced and N600F was returned to service.

Additional evaluation of the incurred instrument component breakdown will be conducted and appropriate future action will be taken. This will be completed by April 2, 1985.

This event occurred in the most limiting plant condition with no significant impact on safety of the unit.

IE 22, 11



Carolina Power & Light Company

Brunswick Steam Electric Plant
P. O. Box 10429
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October 4, 1984

FILE: B09-13510C
SERIAL: BSEP/84-2074

NRC Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555

BRUNSWICK STEAM ELECTRIC PLANT UNIT 1
DOCKET NO. 50-325
LICENSE NO. DPR-71
LICENSEE EVENT REPORT 1-84-22

Gentlemen:

In accordance with Title 10 to the Code of Federal Regulations, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-1022, September 1983.

Very truly yours,

C. R. Dietz, General Manager
Brunswick Steam Electric Plant

MJP/kal/LETKAL

Enclosure

cc: Mr. R. C. DeYoung
Mr. J. P. O'Reilly

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