

LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

AUGUST 1984

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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I. INTRODUCTION

The LaSalle Nuclear Power Station is a Two Unit Facility Located in Marseilles, Illinois. Each Unit is a Boiling Water Reactor with a designed electrical output of 1078 MWe net. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. Unit One is subject to License Number NPF-11, issued on April 17, 1982. The date of initial criticality was June 21, 1982. Unit Two is subject to license number NPF-18, issued on December 16, 1983. The date of initial criticality was March 10, 1984.

This report was compiled by Randy S. Dus telephone number (815)357-6761, extension 324.

II. MONTHLY REPORT FOR UNIT ONE

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

AUGUST 1-31

The Unit started the reporting period at 92% power. At 2300 hours on August 3, power was reduced to 80% power for a high pressure heater out of service. At 0700 hours on August 6, reactor power was increased to 91%. At 1510 hours on August 8, power was reduced to 70% to reduce conductivity and chloride levels in the reactor water. At 1500 hours on August 13, power was increased to 94%. At 0842 hours on August 25, power was decreased to 85% due to a reactor recirc pump trip during the performance of LIS-NB-09. At 0800 on August 27, power was increased to 98%. At 0000 hours on August 31, power was reduced to 81% per the load dispatcher. The reactor was critical for the entire month of August totalling 744 hours.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specification.

Amendment No. 18. - This amendment revised the Unit 1 Technical Specification to reflect the changes that were previously approved for the Unit 2 Technical Specifications. This amendment affects the entire Unit 1 Technical Specification.

2. Facility or procedure changes requiring NRC approval.

There were no facility or procedure changes requiring NRC approval.

3. Tests and Experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval.

4. Corrective maintenance of safety related equipment.

The following table (Table 1) presents a summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request numbers, LER numbers, Component Name, Cause of Malfunction, Results and Effects on Safe Operation, and Corrective Action.

TABLE 1
CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

L7P-300-7
Revision 4
February 29, 1984
6

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L24803		DIV I SWGR Room Vent damper.	Damper actuator Leaking oil and will not position damper.	Damper Still functional manually.	Repaired actuator re- turned to service.
L38733		RHR Heat Ex- changer by- pass vlv.	Stake the setscrew in the Anti-rotation stem to prevent loosening	Preventative Maintenance.	Reinstalled setscrew with locktite.
L39207		Suppression pool temp- erature re- corder.	Chart drive broke paper does not advance	Recorder still prints correctly.	Replaced high speed clutch.
L39374		ADS Continuity monitor	Continuity lights not lit. Potential loss to ads circuit.	All circuits verified to be operable.	Tightened loose socket connection.
L39549		ESF Annun- ciator panel.	Spurious alarm occurrence Which could not be reset.	Computer verified spurious alarm.	Replaced digital logic card.
L39553		D/G Cooling Water pump	Pressure gauge reading low, suspect inadequate cooling to diesel.	Pump pressure gauge verified to be out of calibration.	Recalibrated pressure gauge.
L39619		RHR Supp- ression pool spray vlv.	Valve trips on thermals when going closed.	Fully operational in open direction.	Cleaned torque switch contacts.
L39701		Reactor scram reset switch	Switch does not return fully to the mid position.	Still able to scram reset.	Cam on actuator was sticking, replaced actuator.

TABLE 1
CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

LTP-300-7
Revision 4
February 29, 1984
6

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L39976		RHR service water pump	Bearing assembly assembled wrong.	Redundant service water pumps operable.	Removed bearing housing and re- assembled.
L40341		Rod block monitor	"B" Channel not properly decoding the LPRM string for the rod selected.	None. Redundant channels operable.	Replaced defective multiplexer card.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, occurring during the reporting period, August 1 through August 31, 1984. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
84-045-00		Reactor Water Cleanup High Differential Flow Isolation.
84-046-00		Reactor Water Cleanup High Differential Flow Isolation.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORTDOCKET NO. 050-373UNIT LaSalle OneDATE September 6, 1984COMPLETED BY Randy S. DusTELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: August 1984 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): 3323 MAX DEPEND CAPACITY
(MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	<u>744.0</u>	<u>4746</u>	<u>4746</u>
6. REACTOR RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>1076</u>	<u>1076</u>
7. HOURS GENERATOR ON LINE	<u>744.0</u>	<u>4586</u>	<u>4586</u>
8. UNIT RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>1.0</u>	<u>1.0</u>
9. GROSS THERMAL ENERGY GENERATED (MWH)	<u>2223602</u>	<u>13126524</u>	<u>13126524</u>
10. GROSS ELEC. ENERGY GENERATED (MWH)	<u>713445</u>	<u>4283229</u>	<u>4283229</u>
11. NET ELEC. ENERGY GENERATED (MWH)	<u>677248</u>	<u>4080111</u>	<u>4080111</u>
12. REACTOR SERVICE FACTOR	<u>100%</u>	<u>81.0%</u>	<u>81%</u>
13. REACTOR AVAILABILITY FACTOR	<u>100%</u>	<u>99.4%</u>	<u>99.4%</u>
14. UNIT SERVICE FACTOR	<u>100%</u>	<u>78.3%</u>	<u>78.3%</u>
15. UNIT AVAILABILITY FACTOR	<u>100%</u>	<u>78.3%</u>	<u>78.3%</u>
16. UNIT CAPACITY FACTOR (USING MDC)	<u>87.9%</u>	<u>67.3%</u>	<u>67.3%</u>
17. UNIT CAPACITY FACTOR(USING DESIGN MWe)	<u>84.4%</u>	<u>64.6%</u>	<u>64.6%</u>
18. UNIT FORCED OUTAGE RATE	<u>0.0%</u>	<u>18.0%</u>	<u>18.0%</u>
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)			
On October 1, 1984 there will be a four week outage to inspect the drywell and perform scheduled surveillances.			
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: <u>N/A</u>			
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):			
	FORECAST	ACHIEVED	
INITIAL CRITICALITY	<u> </u>	<u>6/21/82</u>	
INITIAL ELECTRICITY	<u> </u>	<u>9/04/82</u>	
COMMERCIAL OPERATION	<u> </u>	<u>1/1/84</u>	

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-373
 UNIT: LASALLE ONE
 DATE: SEPTEMBER 1984
 COMPLETED BY: Randy S. Dus
 TELEPHONE: (815) 357-6761

MONTH: AUGUST 1984

DAY AVERAGE DAILY POWER LEVEL
 (MWe-Net)

DAY AVERAGE DAILY POWER LEVEL
 (MWe-Net)

1. _____ 920 _____	17. _____ 955 _____
2. _____ 939 _____	18. _____ 943 _____
3. _____ 966 _____	19. _____ 938 _____
4. _____ 733 _____	20. _____ 994 _____
5. _____ 770 _____	21. _____ 1023 _____
6. _____ 916 _____	22. _____ 1026 _____
7. _____ 947 _____	23. _____ 1033 _____
8. _____ 802 _____	24. _____ 1031 _____
9. _____ 778 _____	25. _____ 899 _____
10. _____ 639 _____	26. _____ 921 _____
11. _____ 688 _____	27. _____ 1008 _____
12. _____ 825 _____	28. _____ 1007 _____
13. _____ 932 _____	29. _____ 970 _____
14. _____ 944 _____	30. _____ 956 _____
15. _____ 914 _____	31. _____ 794 _____
16. _____ 969 _____	

INSTRUCTIONS

On this form list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line.) In such cases the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-374
UNIT NAME LaSalle One
DATE September 1984
COMPLETED BY Randy S. Dus
TELEPHONE (815)357-6761

REPORT MONTH AUGUST 1984

NO.	DATE	TYPE	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED S: SCHEDULED				
15	84/08 '04	S	0	H	5	Reduced power to repair steam leaks on high pressure heater

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief valve operations for Unit One.

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO & TYPE ACTUATION</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
8/28/84	1B21-F013S	1 Auto	981 psig	Grounded on "C" Solenoid.
8/29/84	1B21-F013S	4 Auto	981 psig	Grounded on "C" Solenoid.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
1-637-84	HPCS Diesel/Gen and Cooling Water Pump	Change oil in pump
1-654-84	1C RHR Service Water Pump.	Inspect pump bearings.
1-629-84	1A Diesel/Gen	Lubrication System Inspection.

3. Off-Site Dose Calculation Manual

There were no changes to the off-site dose calculations manual during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no changes to the Radioactive Waste Treatment System during this reporting period.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

AUGUST 1984

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

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 - 1. Safety/Relief Valve Operations
 - 2. ECCS System Outages
 - 3. Off-Site Dose Calculation Manual Changes
 - 4. Major Changes to Radioactive Waste Treatment System

I. INTRODUCTION

The LaSalle Nuclear Power Station is a Two Unit Facility Located in Marseilles, Illinois. Each Unit is a Boiling Water Reactor with a designed electrical output of 1078 MWe net. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. Unit One is subject to License Number NPF-11, issued on April 17, 1982. The unit commenced commercial generation of power on January 1, 1984. Unit Two is subject to license number NPF-18, issued on December 16, 1983. The date of initial criticality was March 10, 1984.

This report was compiled by Randy S. Dus, telephone number (815)357-6761, extension 324.

II. MONTHLY REPORT FOR UNIT TWO

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT TWO

August 1-5 The Unit started the reporting period shutdown for steam leak repairs to the moisture separator reheater. At 1200 hours on August 1, the reactor went critical. At 0550 hours on August 2, the main generator was synchronized to the grid. At 0700 hours on August 2, reactor power was raised to 18%. At 1400 hours on August 2, reactor power was raised to 47%. At 0230 hours on August 3, reactor power was decreased to 25% to repair steam leaks on the second stage reheater drain tank. At 2300 hours on August 3, reactor power was raised to 60%. At 1400 hours on August 4, reactor power was raised to 77%. At 1900 hours on August 5, the reactor scrammed due to an unauthorized person lifting leads at the turbine control panel. The reactor was critical for 193 hours.

August 6-14 The reactor went critical at 0830 hours on August 6. At 1605 hours on August 6, the main generator was synchronized to the grid. At 2200 hours on August 6, reactor power was raised to 38%. At 0700 hours on August 7, reactor power was raised to 66%. At 0700 hours on August 9, reactor power was raised to 90%. At 0750 hours on August 10, the reactor scrammed on a closure of a turbine control valve. The reactor was critical for 95 hours and 20 minutes.

August 15-28 At 1200 hours on August 15, the reactor went critical. At 2325 hours on August 15, the main generator was synchronized to the grid. At 0600 hours on August 16, reactor power was raised to 27%, at 1550 hours on August 16, reactor power was raised to 40%. At 0854 hours on August 17, the reactor scrammed due to a main generator exciter failure. The reactor was critical for 44 hours and 54 minutes.

August 29-31 At 2240 hours on August 29, the reactor went critical. At 1723 hours on August 30, the main generator was synchronized to the grid. At 0700 hours on August 31, reactor power was raised to 53%. The reactor was critical for 49 hours and 20 minutes.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED
MAINTENANCE.

1. Amendments to facility license or Technical Specifications.

There were no facility license or Technical Specification
Amendments during the reporting period.

2. Facility or procedure changes requiring NRC approval.

There were no facility or procedure changes requiring NRC
approval during the reporting period.

3. Tests and experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval during
the reporting period.

4. Corrective Maintenance of Safety Related Equipment.

The following table (Table 1) presents a summary of
safety-related maintenance completed on Unit One during the
reporting period. The headings indicated in this summary
include: Work Request numbers, LER Numbers, Component Name,
cause of malfunction, results and effects on safe operation, and
corrective action.

TABLE 1
CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

LTP-300-7
Revision 4
February 29, 1984
6

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L39179		RPV Differential pressure indicator.	Indicator failed downscale	Redundant indication available and operable.	Replaced leaking 3 way manifold.
L38622		Drywell Electrical penetration.	Pressure less than 5#'s. Suspect leak in penetration.	Containment integrity still maintained.	Recharged penetration. No Leaks found.
L38734		RHR heat exchanger bypass vlv.	Stake the setscrew in the Anti-Rotation stem to prevent loosening.	Preventative Maintenance.	Reinstalled set screw with locktite.
L38919		Post Loca gross gamma recorder	Recorder does not drive in slow speed.	Recorder still functional in fast speed.	Replaced chart drive circuit board and motor assembly.
L39019		LPCI Low header pressure alarm.	Low Header Pressure alarm always up.	Verified adequate pressure in system.	Recalibrated pressure switch.
L39714		Suppression pool temperature recorder.	Recorder occasionally stops driving.	Recorder still prints but does not drive. Redundant division still operable.	Replaced pinion gear, motor & clutch.
L40159		RCIC Condenser vacuum pump.	Run Light energized but pump does not run.	Commutator very dirty and slightly burnt.	Cleaned commutator and reseated brushes.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENTLTP-300-7
Revision 4
February 29, 1984
6

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L40218		Main Steam line drain valve	Valve will not seal in closed.	Valve still operable but not able to seal in.	Cleaned torque switch contacts. Verified seal in feature operable.
L40255		Wide range suppression pool level indicator.	Defective amplifier circuit board.	Fails downscale with adequate level present.	Replaced amplifier circuit board.
L40378		LPCS Full flow test valve.	Valve Does not cycle from control room.	Still functional manually.	Cleaned defective torque switch contacts.
L40447		RHR Injection testable check Vlv.	Not able to cycle. Valve due to leaking air operator.	Valve still functional with system flow and pressure.	Installed new air cylinder.
L40340		RHR Full Flow Test Vlv.	Disc. Misalignment to body guide	Redundant RHR Loop still operational	Disc skirt installed

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit Two, occurring during the reporting period, August 1 through August 31, 1984. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
84-036-00		Reactor Water Cleanup High Differential Flow Isolation.
84-037-00		Reactor Water Cleanup High Differential Flow Isolation.
84-038-00		Unsecured High Radiation Area Door.
84-039-00		Reactor Low Pressure RCIC Isolation.
84-040-00		Drywell crane circuits not on Tech Spec Surveillance.
84-041-00		Reactor Water Cleanup High Differential Flow Isolation.
84-042-00		Unit 2 Manual Scram Shutdown
84-043-00		Violation of Tech Spec 3.6.1.8 Action Statement (VQ in Drywell Ventilation)
84-044-00		Reactor Water Cleanup High Differential Flow Isolation.
84-045-00		Low CRD Header Pressure Scram
84-046-00		Reactor Water Cleanup Isolation-Leak Detection High Ambient Temperature.
84-047-00		Scram due to vendor error
84-048-00		Reactor Scram Initiation-Reactor Instrumentation line valved in.
84-049-00		Unsecured high Radiation Area Door.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-374UNIT LaSalle TwoDATE September 1984COMPLETED BY Randy S. DusTELEPHONE (815)357-6761

OPERATING STATUS

1.	REPORTING PERIOD: <u>August 1984</u> GROSS HOURS IN REPORTING PERIOD: <u>744</u>		
2.	CURRENTLY AUTHORIZED POWER LEVEL (MWt): <u>3323</u> MAX DEPEND CAPACITY (MWe-Net): <u>1036</u> DESIGN ELECTRICAL RATING (MWe-Net): <u>1078</u>		
3.	POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): <u>N/A</u>		
4.	REASONS FOR RESTRICTION (IF ANY):		
		THIS MONTH	YR TO DATE
5.	NUMBER OF HOURS REACTOR WAS CRITICAL	<u>292.6</u>	<u>2811</u>
6.	REACTOR RESERVE SHUTDOWN HOURS	<u>451.4</u>	<u>1380</u>
7.	HOURS GENERATOR ON LINE	<u>237.0</u>	<u>1942</u>
8.	UNIT RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>0.0</u>
9.	GROSS THERMAL ENERGY GENERATED (MWH)	<u>475310</u>	<u>3289078</u>
10.	GROSS ELEC. ENERGY GENERATED (MWH)	<u>137874</u>	<u>915324</u>
11.	NET ELEC. ENERGY GENERATED (MWH)	<u>130211</u>	<u>860720</u>
12.	REACTOR SERVICE FACTOR	<u>N/A</u>	<u>N/A</u>
13.	REACTOR AVAILABILITY FACTOR	<u>N/A</u>	<u>N/A</u>
14.	UNIT SERVICE FACTOR	<u>N/A</u>	<u>N/A</u>
15.	UNIT AVAILABILITY FACTOR	<u>N/A</u>	<u>N/A</u>
16.	UNIT CAPACITY FACTOR (USING MDC)	<u>N/A</u>	<u>N/A</u>
17.	UNIT CAPACITY FACTOR (USING DESIGN MWe)	<u>N/A</u>	<u>N/A</u>
18.	UNIT FORCED OUTAGE RATE	<u>N/A</u>	<u>N/A</u>
19.	SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)		
20.	IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP		
21.	UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):		
		FORECAST	ACHIEVED
	INITIAL CRITICALITY		<u>3/10/84</u>
	INITIAL ELECTRICITY		<u>4/20/84</u>
	COMMERCIAL OPERATION	<u>Aug. 84</u>	

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-374
UNIT: LASALLE TWO
DATE: September 1984
COMPLETED BY: Randy S. Dus
TELEPHONE: (815) 357-6761
MONTH: August 1984

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	0
2.	181
3.	287
4.	684
5.	615
6.	98
7.	605
8.	737
9.	934
10.	329
11.	0
12.	0
13.	0
14.	0
15.	1
16.	253

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	152
18.	0
19.	0
20.	0
21.	0
22.	0
23.	0
24.	0
25.	0
26.	0
27.	0
28.	0
29.	0
30.	36
31.	514

INSTRUCTIONS

On this form list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line.) In such cases the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

ATTACHMENT E

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH AUGUST 1984

DOCKET NO. G50-374

UNIT NAME LaSalle Two

DATE September 1984

COMPLETED BY Randy S. Dus

TELEPHONE (815)357-6761

NO.	DATE	TYPE	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED S: SCHEDULED				
25	840805	F	13.5	G	3	Unauthorized person incorrectly connected DVM resulting in a turbine trip.
26	840810	F	124.1	B	3	Closure of #1 turbine control valve resulted in high reactor pressure.
27	240817	F	301.8	A	3	Loss of excitation to U-2 Generator caused a turbine trip.

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief Valve Operations for Unit Two.

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO & TYPE ACTUATIONS</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
8/5/84	2B21-F013D	1 Auto	981 psig	Rx Scram
8/5/84	2B21-F013S	1 Auto	981 psig	Rx Scram.
8/5/84	2B21-F013V	1 Auto	981 psig	Rx Scram.
8/15/84	2B21-F013A	1 Auto	981 psig	SRV Cycling for MOD 1-2-84-120
8/15/84	2B21-F013B	1 Auto	981 psig	SRV Cycling for MOD 1-2-84-120
8/15/84	2B21-F013C	1 Auto	981 psig	SRV Cycling for MOD 1-2-84-120
8/15/84	2B21-F013D	1 Auto	981 psig	SRV Cycling for MOD 1-2-84-120
8/15/84	2B21-F013E	1 Auto	981 psig	SRV Cycling for MOD 1-2-84-120
8/15/84	2B21-F013F	1 Auto	981 psig	SRV Cycling for MOD 1-2-84-120
8/15/84	2B21-F013G	1 Auto	981 psig	SRV Cycling for MOD 1-2-84-120
8/15/84	2B21-F013H	1 Auto	981 psig	SRV Cycling for MOD 1-2-84-120
8/15/84	2B21-F013I	1 Auto	931 psig	SRV Cycling for MOD 1-2-84-120
8/15/84	2B21-F013J	1 Auto	981 psig	SRV Cycling for

E. UNIQUE REPORTING REQUIREMENTS

(CONT'D)

1. Safety/Relief Valve Operations for Unit Two.

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO & TYPE ACTUATIONS</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
8/15/84	2B21-F013K	1 Auto	981 psig	SRV Cycling for MOD 1-2-84-120
8/15/84	2B21-F013L	1 Auto	981 psig	SRV Cycling for MOD 1-2-84-120
8/15/84	2B21-F013M	1 Auto	981 psig	SRV Cycling for MOD 1-2-84-120
8/15/84	2B21-F013N	1 Auto	981 psig	SRV Cycling for MOD 1-2-84-120
8/15/84	2B21-F013P	1 Auto	981 psig	SRV Cycling for MOD 1-2-84-120
8/15/84	2B21-F013R	1 Auto	981 psig	SRV Cycling for MOD 1-2-84-120
8/15/84	2B21-F013S	1 Auto	981 psig	SRV Cycling for MOD 1-2-84-120
8/15/84	2B21-F013U	1 Auto	981 psig	SRV Cycling for MOD 1-2-84-120
8/15/84	2B21-F013V	1 Auto	981 psig	SRV Cycling for MOD 1-2-84-120

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
2-948-84	2B DG Air Compressor	Replace Gasket
2-971-84	2E12-F322A/B	STP 71
2-981-84	HPCS D/G Air Start Motor Comp.	Replace Relief
2-997-84	2E12-F024B	Troubleshoot M.O.
2-998-84	2E12-F024B	Troubleshoot M.O.
2-1002-84	2E12-F024B	Repair Valve
2-1010-84	2E12-D300B/RHR SW Strainer	Repair Strainer Gasket/Seal.
2-1012-84	SF & RHR	Install SF Spool Pieces.
2-1039-84	HPCS D/G	18 Month Inspection
2-1049-84	B RHR W/L Pump	Lubrication.
2-1050-84	2A D/G	18 Month Surv.
2-1051-84	2A D/G Air Syst.	Repair Leaking Valve.
2-1056-84	U-2 LPCS	Lube.
2-1073-84	Suppression Pool Cleanup	Remove Spool Pieces.
2-1087-84	2E12-F041A	Repair Operator
2-1090-84	2E12-F05A	Troubleshoot & Repair
2-1092-84	2E12-F074A	T.S. Action Item.
2-1098-84	2E12-F003B	Set Limit SW's.

3. Off-Site Dose Calculation Manual

There were no changes to the off-site dose calculations manual during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no changes to the Radioactive Waste Treatment System during this reporting period.



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

September 10, 1984

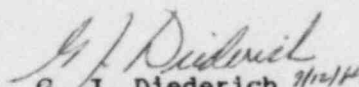
Director, Office of Management Information
and Program Control
United States Nuclear Regulatory Commission
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering
LaSalle County Nuclear Power Station for the period covering August 1 through
August 31, 1984.

Very truly yours,


G. J. Diederich 9/12/84
Superintendent
LaSalle County Station

GJD/RSD/crh

Enclosure

xc: J. G. Keppler, NRC, Region III
NRC Resident Inspector LaSalle
Gary Wright, Ill. Dept. of Nuclear Safety
D. P. Galle, CECO
D. L. Farrar, CECO
INPO Records Center
Ron A. Johnson, PIP Coordinator SNED
W. R. Jackson, GE Resident
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