

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Prairie Island Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 2 8 2				PAGE (3) 1 OF 2		
TITLE (4) Reactor Trip on Startup																
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)			
08	31	84	84	006	00	09	28	84					0 5 0 0 0			
OPERATING MODE (9) N			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11)													
POWER LEVEL (10) 0 2 0			20.402(b)				20.405(a)				<input checked="" type="checkbox"/> 90.73(a)(2)(iv)				73.71(b)	
			20.405(a)(1)(i)				90.36(a)(1)				<input type="checkbox"/> 90.73(a)(2)(v)				73.71(e)	
			20.405(a)(1)(ii)				90.36(a)(2)				<input type="checkbox"/> 90.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 388A)	
			20.405(a)(1)(iii)				90.73(a)(2)(i)				<input type="checkbox"/> 90.73(a)(2)(viii)(A)					
			20.405(a)(1)(iv)				90.73(a)(2)(ii)				<input type="checkbox"/> 90.73(a)(2)(viii)(B)					
			20.405(a)(1)(v)				90.73(a)(2)(iii)				<input type="checkbox"/> 90.73(a)(2)(ix)					
LICENSEE CONTACT FOR THIS LER (12)																
NAME Arne A. Hunstad, Staff Engineer										TELEPHONE NUMBER AREA CODE 6 1 2 3 8 8 - 1 1 2 1						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC						
X	S	J	FT	F 1 8 0	Y											
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO				
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)																

During trip recovery, the reactor tripped at about 20% power on low steam generator level plus steam flow/feedwater flow mismatch. Faulty feedwater flow transmitter was recalibrated and will be replaced.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (8)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Prairie Island Unit 1	0 5 0 0 0 2 8 2 8 4	— 0	0 6	— 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 365A's) (17)

During recovery from the reactor trip of 8-30-84, reactor power was at about 20%. The Loop A feedwater (SJ) flow channel had drifted low resulting in a steam flow/feedwater flow mismatch indication. Difficulty in controlling feedwater flow at this low power level was resulting in some steam generator level swings. At 0002 on August 31, 1984, the mismatch indication and a low steam generator level indication occurred together, satisfying the logic for reactor trip.

The feedwater flow transmitter (FT) had drifted out of calibration. The transmitter, a Foxboro D/P Cell Electronic Transmitter Model E13DH Style B, was recalibrated and unit restart was successful. A similar event involving feedwater flow controllability is described in RE 84-1 dated January 3, 1984. It is planned that feedwater regulating valve (FCV) trim be replaced at the next refueling. The faulty feedwater flow transmitter is being observed closely and will be replaced at the first opportunity. Feasibility of improving feedwater flow indication in the low flow range is being studied. Operators will practice controlling steam generator level under similar conditions on the simulator.



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U S Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
Docket No. 50-282 License No. DPR-42
50-306 DPR-60

Reactor Trip On Startup

The License Event Report for this occurrence is attached.

This event was reported via Emergency Notification System per 10 CFR Part 72
on August 31, 1984.

Eugene Eckholt

for David Musolf
Manager - Nuclear Support Services

DMM/EFE/dab

c: Regional Administrator-III, NRC
NRR Project Manager, NRC
Resident Inspector, NRC
MPCA
Attn: J W Ferman

Attachment

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