

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2										DOCKET NUMBER (2) 0 5 0 0 0 3 6 1				PAGE (3) 1 OF 0 2		
TITLE (4) TWO INOPERABLE SAFETY INJECTION TANKS																
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQ. NUMBER	REV. NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)			
0 8	2 9	8 4	8 4	0 5 1	0 0	0 9	2 8	8 4					0 5 0 0 0 0			
OPERATING MODE (9) 1		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)														
POWER LEVEL (10) 1 0 0		20.402(b)				20.405(c)				50.73(a)(2)(iv)				73.71(b)		
		20.405(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(c)		
		20.405(a)(1)(ii)				X 50.36(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)		
		20.405(a)(1)(iii)				X 50.73(a)(2)(i)				50.73(a)(2)(viii)(A)						
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)						
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(x)						
LICENSEE CONTACT FOR THIS LER (12)																
NAME J. G. HAYNES, STATION MANAGER										TELEPHONE NUMBER 7 1 4 4 9 2 1 - 7 1 7 0 1 0						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC						
X	B P	IV T IV	T 0 2 0	N												
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)												X NO				

Abstract (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 8/29/84 at 2154, with Unit 2 in Mode 1 at 100 percent power, while filling Safety Injection Tank (SIT) No. 10, the nitrogen cover pressure on SIT No. 7 increased to 630 psig due to leakage into the tank through its fill and drain valve. Since vent valve fuses had been installed on SIT No. 10, and since the nitrogen cover pressure on SIT No. 7 exceeded the limits of Limiting Condition for Operation (LCO) 3.5.1, two SITs were inoperable, and LCO 3.0.3 was invoked. The vent valve fuses were removed from SIT No. 10 at 2159, thereby exiting LCO 3.0.3, and installed on SIT No. 7. However, the vent valve for SIT No. 7 would not operate. To reduce pressure on SIT No. 7, the level was lowered by draining to the Reactor Coolant Drain Tank. At 2204 leakage past SIT No. 10's fill and drain valve caused its level to fall and its cover pressure decreased to 595 psig, less than the minimum specified in LCO 3.5.1, and LCO 3.0.3 was invoked. At approximately 2211, the pressure in SIT's No. 7 and No. 10 was restored to within the limits of LCO 3.5.1, and LCO 3.0.3 was exited.

The vent valve for SIT No. 7 will be repaired during an outage. The present design of the SIT fill and drain valves is inadequate to provide positive isolation under a high differential pressure. Therefore, the SIT fill and drain valves will be replaced with valves of a different design pending their procurement.

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LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQ. NUMBER	REV. NUMBER			
SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2	05000361	84	051	00	02	OF	02

TEXT (If more space is required, use additional NRC Form 366A's) (17)

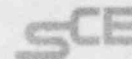
On August 29, 1984, at 2154, with Unit 2 in Mode 1 at 100 percent power, during the performance of the procedure for filling Safety Injection Tank (EIIIS Component Identifier TK) No. 10, the nitrogen cover pressure on Safety Injection Tank No. 7 increased to 630 psig due to leakage into the tank through its fill and drain valve (EIIIS Component Identifier ISV). Since vent valve fuses had been installed on Tank No. 10, and since the cover pressure on Tank No. 7 exceeded the limits of Limiting Condition for Operation (LCO) 3.5.1, two Safety Injection Tanks were inoperable and LCO 3.0.3 was invoked. The vent valve fuses were removed from Tank No. 10 at 2159, thereby exiting 3.0.3, and installed in Tank No. 7. However, the vent valve (EIIIS Component Identifier VTV) for Tank No. 7 would not open. Operators therefore reduced pressure in Tank No. 7 by draining water from the tank. Leakage past Tank No. 10's fill and drain valve caused its level and pressure to also decrease. At 2204 pressure on Tank No. 10 was reduced to less than the minimum specified in LCO 3.5.1 at 595 psig, and LCO 3.0.3 was invoked. At approximately 2211, the pressure in Tanks No. 7 and No. 10 was restored to within limits, and LCO 3.0.3 was exited.

The vent valve for Tank No. 7 will be repaired during an outage. The present design of the Safety Injection Tank fill and drain valves is inadequate to provide positive isolation under a high differential pressure. The Safety Injection Tank fill and drain valves will be replaced with valves of a different design pending their procurement. The Safety Injection Tank fill and drain procedure was revised to provide precautions to be taken until the fill and drain valves are replaced.

LCO 3.5.1 requires that the nitrogen cover pressure on the Safety Injection Tanks be maintained between 600 and 625 psig. The two deviations between these required conditions and the actual conditions were small, and the time of the deviations were only five and seven minutes, respectively. Therefore, there was no significant loss of safety function during this event.

There are no reasonable alternative conditions under which this event would have been more severe.

Southern California Edison Company



SAN ONOFRE NUCLEAR GENERATING STATION

P.O. BOX 128

SAN CLEMENTE, CALIFORNIA 92672

J. G. HAYNES
STATION MANAGER

September 28, 1984

TELEPHONE
(714) 492-7700

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Docket No. 50-361
30-Day Report
Licensee Event Report No. 84-051
San Onofre Nuclear Generating Station, Unit 2

Pursuant to 10 CFR 50.36(c)(2) and 50.73(a)(2)(i)(B), this submittal provides the required 30-day written Licensee Event Report (LER) for an occurrence involving Limiting Condition for Operation 3.5.1. Neither the health and safety of plant personnel nor the public were affected by this event.

If you require any additional information, please so advise.

Sincerely,

JG Haynes/HEM

Enclosure: LER 84-051

cc: A. E. Chaffee (USNRC Senior Resident Inspector, Units 1, 2 and 3)
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

J. B. Martin (Regional Administrator, NRC Region V)

Institute of Nuclear Power Operations (INPO)

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