NRC Form 366
(9/83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 3	0 5 0 0 0 3 6 2	1 OF 0 2

TITLE (4)
STEAM GENERATOR WIDE RANGE CHANNEL INOPERABLE

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQ. NUMBER	REV. NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 8	2 3	8 4	8 4	0 3 6	0 0	0 9	2 4	8 4		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)	3	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 0 0	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
J. G. HAYNES, STATION MANAGER	7 1 4 — 4 9 2 - 7 7 0 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	I P	L I	S 1 8 5	N							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

Abstract (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 8/16/84 at 0100, during an unrelated surveillance, an Assistant Control Operator (ACO) noted that Steam Generator Wide Range Level Indicator 3LI-1125-2 (EIIS Component Identifier LI) had failed low. However, because the instrument was not uniquely identified as part of Accident Monitoring Instrumentation (AMI), the ACO did not recognize that it had to be restored within seven days. The ACO prepared a routine maintenance request instead of an accelerated maintenance request. This condition was observed at 1920 on 8/23/84 with the unit in Mode 3 and cooldown to Mode 4 was initiated in accordance with Limiting Condition for Operation 3.3.3.6. At 2000, on 8/23/84, 3LI-1125-2 was returned to service following replacement of a faulty lumnigraph assembly and cooldown to Mode 4 was terminated.

As corrective action, all AMI has been labeled. Additionally, the significance of this event, the labeling of AMI, and the use of accelerated maintenance requests were discussed at shift briefings.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQ. NUMBER	REV. NUMBER	
SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 3	0 5 0 0 0 3 6 2	8 4	– 0 3 6	– 0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 8/23/84, with Unit 3 in Mode 3, a routine equipment control review of maintenance orders was in progress. At 1920, during this review, Steam Generator Wide Range Level Indicator 3LI-1125-2 (EIIS Component Identifier LI) was discovered to have been inoperable since 0100 on 8/16/84, contrary to Limiting Condition for Operation (LCO) 3.3.3.6, Action Statement 20. This Action Statement requires restoration of the inoperable indicator within 7 days or be in Mode 4 within the next 12 hours. Cooldown to Mode 4 was initiated and corrective maintenance 3LI-1125-2 was commenced. At 2000, on 8/23/84, 3LI-1125-2 was returned to service following replacement of a faulty lumnigraph assembly and cooldown to Mode 4 was terminated.

On 8/16/84 during an unrelated surveillance, an Assistant Control Operator (ACO) noted that 3LI-1125-2 had failed low. However, because the instrument was not uniquely identified as part of Accident Monitoring Instrumentation (AMI), the ACO did not recognize that it had to be restored within seven days. The ACO prepared a routine maintenance request instead of an accelerated maintenance request. As corrective action, all AMI has been labeled. Additionally, the significance of this event, the labeling of AMI, and the use of accelerated maintenance requests were discussed at shift briefings.

Redundant monitor 3LI-1125-1 remained operable throughout this event. There are no reasonable or credible alternative circumstances under which this event would have been more severe.

Southern California Edison Company

SAN ONOFRE NUCLEAR GENERATING STATION
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SAN CLEMENTE, CALIFORNIA 92672

J. G. HAYNES
STATION MANAGER

TELEPHONE
(714) 492-7700

September 24, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Docket No. 50-362
30-Day Report
Licensee Event Report No. 84-036
San Onofre Nuclear Generating Station, Unit 3

Pursuant to 10 CFR 50.73(a)(2)(i)(B), this submittal provides the required 30-day written Licensee Event Report (LER) for an occurrence involving Accident Monitoring Instrumentation. Neither the health and safety of the public nor plant personnel were affected by this event.

If you require any additional information, please so advise.

Sincerely,

JGHaynes

Enclosure: LER No. 84-036

cc: A. E. Chaffee (USNRC Senior Resident Inspector, Units 1, 2 and 3)
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

J. B. Martin (Regional Administrator, NRC Region V)

Institute of Nuclear Power Operations (INPO)

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