

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Washington Nuclear Plant - Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 3 9 17										PAGE (3) 1 OF 0 12													
TITLE (4) Containment Temperature Monitoring																																	
EVENT DATE (5)						LER NUMBER (6)						REPORT DATE (7)						OTHER FACILITIES INVOLVED (8)															
MONTH		DAY		YEAR		YEAR		SEQUENTIAL NUMBER		REVISION NUMBER		MONTH		DAY		YEAR		FACILITY NAMES				DOCKET NUMBER(S)											
																						0 5 0 0 0											
0 9		1 8		8 4		4 8		4 0		3 4		0 3		1 0		0 4						0 5 0 0 0											
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																															
★		20.402(h)										20.406(e)										50.73(a)(2)(iv)										73.71(b)	
POWER LEVEL (10)		20.406(a)(1)(i)										50.38(e)(1)										50.73(a)(2)(v)										73.71(c)	
★		20.406(a)(1)(ii)										50.38(e)(2)										50.73(a)(2)(vii)										X OTHER (Specify in Abstract below and in Text, NRC Form 388A)	
		20.406(a)(1)(iii)										50.73(a)(2)(i)										50.73(a)(2)(viii)(A)											
		20.406(a)(1)(iv)										50.73(a)(2)(ii)										50.73(a)(2)(viii)(B)											
		20.406(a)(1)(v)										50.73(a)(2)(iii)										50.73(a)(2)(ix)											
Special Report Tech. Spec. 3.7.8.a																																	

LISCENSEE CONTACT FOR THIS LER (12)										TELEPHONE NUMBER											
NAME R. L. Koenigs, Compliance Engineer										AREA CODE 510 1931 71-12151011											
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										Ext. 2279											

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC					
B	V	B	-	-	-	-	-	N						
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
X YES (If yes, complete EXPECTED SUBMISSION DATE)										NO		11	2	011814

ABSTRACT (Limit to 400 words - a space - one - fifteen single-space typewritten lines) (16)

During Plant heatups, Primary Containment Monitoring System (CMS) temperature indicators exceeded 150°F on 4/15/84, 4/28/84, 7/3/84, 8/4/84 and 9/18/84. The highest temperature attained was 165°F on 4/15/84. During each event the average drywell temperature did not exceed 135°F (as per Technical Specification 3.7.8.a). The 9/18/84 condition is a continuation of the original problem addressed in Revisions 0, 1 and 2 of this LER.

★	Date	Operating Mode	Power Level	LER No.
Event 1	4/15/84	2	001	84-034 Rev. 0
Event 2	4/28/84	2	001	84-034 Rev. 0
Event 3	7/3/84	1	035	84-034 Rev. 1
Event 4	8/4/84	1	050	84-034 Rev. 2
Event 5	9/18/84	1	045	84-034 Rev. 3

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
Washington Nuclear Plant - Unit 2	0 5 0 0 0 3 9 7 8 4	—	0 8 1 4	—	0 3 0	12 OF 0 12

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Plant Condition

Event 1 Plant Mode - 1 Power Level - 1%
Event 2 Plant Mode - 1 Power Level - 1%
Event 3 Plant Mode - 1 Power Level - 35%
Event 4 Plant Mode - 1 Power Level - 50%
Event 5 Plant Mode - 1 Power Level - 45%

Event

On September 18, 1984 Primary Containment Atmospheric Monitoring System (CMS) temperature indicators exceeded 150°F for over eight hours during a return to power operation following cooling system modifications. The high temperatures existed in two of the five monitored upper head areas (CMS-TI-26 and CMS-TI-32) and the highest temperature reached was 158°F. This is reportable per Technical Specification 3.7.8.a.

Immediate Corrective Action

The Technical Staff System Engineer evaluated the conditions prior to reaching the 150°F limit. Based on this evaluation, he directed a change in Primary Containment Cooling System temperature controllers. Plant conditions and the time response were such that the over temperature condition existed for approximately 18 hours.

Further Corrective Action

Modifications to the design of the drywell heating and ventilation have been made. Further modifications are planned and future field changes may be required. A final report on the modifications, and results achieved will be provided later.

Safety Significance

There are a limited number of Class 1 components in the areas of concern. The time and size of the over temperature conditions will have negligible effect when compared to the temperature aging effects experienced over the normal forty year operating lifetime. Thus these localized high temperature events have not jeopardized plant safety or that of the public.

Washington Public Power Supply System

P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

Docket No. 50-397

October 4, 1984

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: NUCLEAR PLANT NO. 2
LICENSEE EVENT REPORT NO. 84-034-03

Dear Sir:

Transmitted herewith is Licensee Event Report No. 84-034-03 for WNP-2 Plant. This report is a special report and is submitted in accordance with the requirements of the WNP-2 Technical Specification Section 6.9.2.

Very truly yours,

JM Powers for

J. D. Martin (M/D 927M)
WNP-2 Plant Manager

JDM:DK:mm

Enclosure:

Licensee Event Report No. 84-034-03

cc: Mr. John B. Martin, NRC - Region V
Mr. A. D. Toth, NRC - Site (901A)
Ms. Dottie Sherman, ANI
INPO Records Center - Atlanta, GA

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