

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Sequoyah, Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 3 2 8				PAGE (3) 1 OF 0 2	
---------------------------------------	--	--	--	--	--	--	--	--	--	--------------------------------------	--	--	--	----------------------	--

TITLE (4) Reactor Trip On Low-Low Steam Generator Level															
--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)														
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES			DOCKET NUMBER(S)											
0	9	0	6	8	4	0	1	7	0	0	1	0	0	5	8	4	0	5	0	0	0		

OPERATING MODE (9) 1		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																					
POWER LEVEL (10) 0 2 2	20.402(b)	20.406(a)(1)(i)	20.406(a)(1)(ii)	20.406(a)(1)(iii)	20.406(a)(1)(iv)	20.406(a)(1)(v)	20.406(c)	50.36(e)(1)	50.36(e)(2)	50.73(a)(2)(i)	50.73(a)(2)(ii)	50.73(a)(2)(iii)	XX 50.73(a)(2)(iv)	50.73(a)(2)(v)	50.73(a)(2)(vi)	50.73(a)(2)(vii)(A)	50.73(a)(2)(vii)(B)	50.73(a)(2)(ix)	73.71(b)	73.71(e)	OTHER (Specify in Abstract below and in Text, NRC Form 306A)		

LICENSEE CONTACT FOR THIS LER (12)										TELEPHONE NUMBER					
NAME Heyward R. Rogers, Compliance Section Engineer										AREA CODE 6 1 5 8 7 0 - 6 1 4 6					

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS

SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE) XX NO														

ABSTRACT (Limits to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During startup following recovery from a reactor trip (see SQRO-50-328/84015) unit 2 experienced another reactor trip. Just prior to the event which occurred at 1000C on 09/06/84, unit 2 was in mode 1 (2235 psi, 551°F) at 22 percent reactor power. The balance of plant operator was in the process of switching control of steam generator levels from manual to automatic. During the switchover the number three (3) steam generator level began increasing and at 60 percent level feedwater was isolated to loop 3. The reactor operator reduced reactor power in an attempt to slow the swell of the number 3 steam generator. With reactor power at 18 percent the steam generator level shrink due to the feedwater isolation resulted in a low-low level in the number four (4) steam generator, which tripped the reactor. The unit stabilized at 547°F following the reactor trip. All equipment and personnel performed and responded as expected following the reactor trip.

8410100264 841C05
PDR ADOCK 05000328
S PDR

IE22 11

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Sequoyah, Unit 2	0 5 0 0 0 3 2 8 8 4	—	0 1 7	—	0 0 0	2	OF 0 2

TEXT (If more space is required, use additional NRC Form 365A's) (17)

With unit 2 in mode 1 at 22 percent reactor power, following a previous trip which occurred on 09/05/84 (reference LER SQRO-50-328/84015), a reactor trip occurred at 1000C on 09/06/84. Prior to the event, with control rods in manual, the balance of plant operator was maintaining steam generator levels by manually controlling the "A" main feedpump and manually manipulating feedwater flow to the steam generators using the bypass regulator valves. Simultaneously, the main turbine was rolled to 1800 RPM and held. During an attempt to switch the feedwater flow from the bypass valves to the main regulator valves, steam generator loop 3 level increased to 60 percent and automatically isolated feed flow to the loop 3 steam generator; however, level continued to increase to approximately 70 percent and the reactor operator stepped control rods in to limit the level transient in loop 3. At the same time loop 3 was increasing, loop 4 was decreasing. The steam generator blowdown to loop 4 was isolated and feedwater flow increased to maintain level; however, level continued to decrease and a reactor trip occurred on low-low steam generator level from loop 4.

Upon receiving the reactor trip, all systems actuated as designed and Operations complied with AOI-1, "Reactor Trip". After an evaluation of the event, it was determined that the unit was safe for restart and the unit returned critical at 1215C. At 1419C the turbine was rolled to speed and at 1535C the turbine tripped on high-high steam generator level. In an attempt to prevent an anticipated reactor trip, the operator inserted the control rods. The trip was prevented; however, in doing so, the low-low rod insertion limit was exceeded and the unit entered LCO 3.1.3.6 at 1540C. Action statement "a" was complied with and the operator began borating to bring rods back out above the insertion limit. The unit was out of the LCO at 1544C. Steam generator levels were stabilized, the turbine restarted, and the generator tied on line at 1609C. The unit proceeded up in power without further incident.

For 1984 this has been the fourth automatic reactor trip for unit 2 and the third automatic reactor trip on low-low steam generator level for unit 2. All equipment and personnel performed as expected during the event and there was no effect on public health and safety.

TENNESSEE VALLEY AUTHORITY

Sequoyah Nuclear Plant
Post Office Box 2000
Soddy Daisy, Tennessee 37379

October 5, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

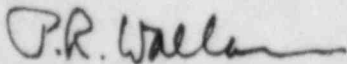
Gentlemen:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNIT 2 - DOCKET NO.
50-328 - FACILITY OPERATING LICENSE DPR-79 - REPORTABLE OCCURRENCE REPORT
SQRO-50-328/84017

The enclosed licensee event report provides details concerning a reactor trip on September 6, 1984, from low-low steam generator level. This event is reported in accordance with 10 CFR 50.73, paragraph a.2.iv.

Very truly yours,

TENNESSEE VALLEY AUTHORITY



P. R. Wallace
Plant Manager

Enclosure
cc (Enclosure):

James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30323

Records Center
Institute of Nuclear Power Operations
Suite 1500
1100 Circle 75 Parkway
Atlanta, Georgia 30339

NRC Inspector, NUC PR, Sequoyah

IE22
1/1