

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) LaSalle County Station Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 3 7 4										PAGE (3) 1 OF 0 3																																
TITLE (4) Reactor Scram																																																				
EVENT DATE (5)										LER NUMBER (6)										REPORT DATE (7)										OTHER FACILITIES INVOLVED (8)																						
MONTH			DAY			YEAR				YEAR			SEQUENTIAL NUMBER			REVISION NUMBER				MONTH			DAY			YEAR				FACILITY NAMES										DOCKET NUMBER(S)												
0 6			0 6			8 4				8 4			0 2			5				0 0			0 6			2 1			8 4														0 5 0 0 0									
OPERATING MODE (9) 1										THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																																										
POWER LEVEL (10) 0 1 2 4										20.402(b)										20.406(c)										X 60.73(a)(2)(iv)										73.71(b)												
										20.405(a)(1)(i)										60.36(a)(1)										60.73(a)(2)(iv)										73.71(c)												
										20.406(a)(1)(ii)										60.36(a)(2)										60.73(a)(2)(iv)										OTHER (Specify in Abstract below and in Text, NRC Form 366A)												
										20.406(a)(1)(iii)										60.73(a)(2)(ii)										60.73(a)(2)(viii)(A)																						
										20.405(a)(1)(iv)										60.73(a)(2)(ii)										60.73(a)(2)(viii)(B)																						
20.406(a)(1)(iv)										60.73(a)(2)(iii)										60.73(a)(2)(ix)																																
LICENSEE CONTACT FOR THIS LER (12)																																																				
NAME Kenneth J. Kalmon, extension 325																				TELEPHONE NUMBER AREA CODE 8 1 1 5 3 1 5 1 7 1 - 1 6 1 7 6 1 1																																
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																																				
CAUSE			SYSTEM			COMPONENT				MANUFACTURER			REPORTABLE TO NRC				CAUSE			SYSTEM			COMPONENT				MANUFACTURER			REPORTABLE TO NRC																						
A			J E			Z 9 9 9				Z 9 9 9			N																																							
SUPPLEMENTAL REPORT EXPECTED (14)																																																				
YES (If yes, complete EXPECTED SUBMISSION DATE)																				X NO										EXPECTED SUBMISSION DATE (15)										MONTH DAY YEAR												
ABSTRACT (Limit to 1400 spaces; i.e., approximately fifteen single-spaced typewritten lines) (16)																																																				
<p>On June 6, 1984, an Instrument Mechanic was in the process of testing Unit 2 Reactor Water Level 2 Level Indicating Transmitter switch when he accidentally vented the variable-side of the instrument. This sent a pressure surge through the instrument lines and caused several other reactor water level switches on the same panel to actuate. These switches tripped both Reactor Recirculation pumps, initiated a Reactor Core Isolation Cooling (RCIC) injection, and gave a Reactor Protection System (RPS) half scram trip. The RCIC injection tripped the Main Turbine and the Main Turbine trip initiated another RPS half scram trip. A full reactor scram resulted.</p> <p>Instrument Maintenance personnel were trained on this event and made aware of the consequences of personnel errors. The need to eliminate personnel error was again stressed.</p>																																																				
JE22 1/1																																																				
8407090252 840621 PDR ADOCK 05000374 S PDR																																																				

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

FACILITY NAME (1) LaSalle County Station Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 3 7 4	LER NUMBER (8)			PAGE (3)		
		YEAR 8 4	SEQUENTIAL NUMBER 0 2 5	REVISION NUMBER 0 0 0	2	OF	0 3

TEXT (If more space is required, use additional NRC Form 306A's) (17)

I. EVENT DESCRIPTION

On June 6, 1984, at 0530 hours with LaSalle Unit 2 at 24% power, several instruments on the 2H22-P027 instrument rack (JL) actuated when Instrument Maintenance personnel were testing Level Indicating Transmitter Switch (LITS) 2B21-N026B (JM) under Work Request L37424. The instruments that actuated caused Reactor Recirculation (AD, RR) pumps 2A and 2B to trip, Reactor Core Isolation Cooling (BN, RCIC) to inject, and a Reactor Protection System (JC, RPS) half scram trip on Channel B1.

The RCIC injection caused a Main Turbine (TA) trip. When the Main Turbine Stop Valves reached the 95% open position, a RPS half scram trip signal on Channel A1 was received, and a full reactor scram occurred.

II. CAUSE

At the time of the occurrence an Instrument Mechanic had just isolated instrument LITS 2B21-N026B, Low-Low Reactor Water Level Transmitter/Switch and opened the instrument's equalizing valve. He was in the process of opening the vent valve on the variable side of the instrument when he accidentally bumped open the variable-side isolation valve. (The two valves are approximately three inches apart.) He quickly closed the isolation valve, but the Control Room alarm printout shows that a RCIC injection occurred very soon after the instrument's isolation valve was bumped open. A Main Turbine trip and full reactor scram followed within two seconds.

The level switches on panel 2H22-P027 use differential pressure to sense water level. The venting of the variable side of instrument LITS-2B21-N026B caused instruments LIS-2B21-N037B and D and LIS-2B21-N036C and D to see a low pressure on their variable side and the switches actuated. The bumping open of the LITS-2B21-N026B isolation valve apparently also sent a pressure transient through the instrument's reference line causing instrument LIS-2B21-N024B to actuate. LIS-2B21-N037B and D (Reactor Vessel Low Low Level 2) initiated a RCIC injection, LIS-2B21-N036C and D (ATWS Trip) tripped RR pumps 2A and 2B, and LIS-2B21-N024B initiated a RPS half scram trip on Channel B1 (Reactor Vessel Low Level 3).

The opening of the RCIC Injection Valve (2E51-F013) caused a Main Turbine trip. When Main Turbine Stop Valve #2 reached the 95% open position a RPS half scram trip on Channel A1 was initiated, completing the full scram signal.

It is allowable to have the Reactor half scram trip from stop valve closure bypassed up to 30% power. This half scram bypass had already dropped out at 24% power. This lower setpoint is conservative but within acceptable limits.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (6)

PAGE (3)

LaSalle County Station Unit 2

0	5	0	0	0	3	7	4	8	4	-	0	2	5	-	0	0	0	3	OF	0	3
---	---	---	---	---	---	---	---	---	---	---	---	---	---	---	---	---	---	---	----	---	---

TEXT (If more space is required, use additional NRC Form 308A's) (17)

III. PROBABLE CONSEQUENCES OF THE OCCURRENCE

The venting of the variable side of instruments on panel 2H22-P027 simulated a low reactor water level condition. All instruments actuated and trips occurred as expected for low level conditions. The plant was brought to a safe shutdown.

IV. CORRECTIVE ACTIONS

Instrument Maintenance personnel were informed of this scram and made aware of the consequences of errors made when working on instruments located on the 2H22-P027 panel. The need to eliminate personnel error was stressed.

V. PREVIOUS OCCURRENCES

No similar occurrences have happened within the year prior to the June 6, 1984 occurrence.

VI. NAME AND TELEPHONE NUMBER OF PREPARER

Kenneth J. Kalmon, 815/357-6761, extension 325.



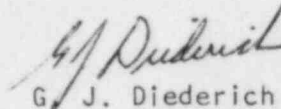
Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

June 21, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

Reportable Occurrence Report #84-025-00, Docket #050-374 is being submitted to your office in accordance with 10 CFR 50.73.


G. J. Diederich
Superintendent
LaSalle County Station

GJD/MLD/kg

Enclosure

xc: NRC, Regional Director
INPO-Records Center
File/NRC

IE22
11