



GPU Nuclear Corporation
Post Office Box 388
Route 9 South
Forked River, New Jersey 08731-0338
609 971-4000
Writer's Direct Dial Number:

June 27, 1984

Mr. Dennis M. Crutchfield
Operating Reactors Branch #5
Division of Licensing
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Dear Mr. Crutchfield:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
NUREG 0737 Status
Re: Full Term Operating License

The purpose of this submittal is to provide you with the current status of NUREG-0737 items for the Oyster Creek plant.

Attached is Table 1, which includes a listing of 0737 items the NRC considers acceptable and resolved.

In addition, Table 2 provides you with the status or target dates of planned modifications.

Should you require any further information on this topic please contact Mr. Michael Laggart, BWR Licensing Manager at (201)299-2341.

Very truly yours,

Peter B. Fiedler
Vice President and Director
Oyster Creek

PBF:BH:dam
Attachments

cc: Dr. Thomas E. Murley, Administrator
Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

NRC Resident Inspector
Oyster Creek Nuclear Generating Station
Forked River, NJ 08731

8407090098 840627
PDR ADOCK 05000219
P PDR

A046
1/1

0737 ITEMS COMPLETED

Table 1

<u>0737 Item</u>	<u>Title</u>	<u>NRC Closeout Date</u>
1.A 1.1.	Shift Technical Advisor	1/15/82
1.A 1.2	Shift Supervisor Responsibilites	5/8/80
1.A 1.3	Shift Manning	2/1/82
1.A 2.1	Upgrading of RO and SRO Training	11/30/82
1.A 2.3	Instructor Complete SRO Exam	*
1.A 3.1	Revise Scope and Criteria for Licensing Exams	*
1.C.2	Shift Turnover Procedures	5/8/80
1.C.3	Shift Supervisor Responsibilities	5/8/80
1.C.4	Control Room Access	5/8/80
1.C.5	Feedback of Operating Experience	2/1/82
1.C.6	Correct Performance of Operating Activities	2/1/82
II.B.4	Training for Mitigating Core Damage	11/30/82
II.D.1	Safety and Relief Valve Testing	6/19/84
II.E.4.2(6)	Containment Vent & Purge Valves	8/31/83
II.E.4.2(7)	Containment Isolation on High Radiation	8/31/83
II.F.1.4	Containment Pressure Monitor	9/29/83
II.F.1.5	Containment Water Level Monitor	9/29/83
II.F.1.6	Containment Hydrogen Monitor	9/29/83
II.K.3.3	Reporting Relief & Safety Valve Failing and Challenges	6/10/82
II.K.3.14	Isolation Condenser Isolation Modification	12/18/81
II.K.3.16	Reduction of Challenges and Failures to Relief Valves	5/8/80

<u>0737 Item</u>	<u>Title</u>	<u>NRC Closeout Date</u>
II.K.3.17	Report on Outages of ECC Systems	8/4/83
II.K.3.21	Core Spray Modifications	5/27/83
II.K.3.25	Effects of Loss of A-C Power on Pump Seals	10/22/82
II.K.3.29	Study to Demonstrate Performance of Isolation Condensers with Noncondensibles	8/15/81
II.K.3.30	Small Break LOCA Model	12/21/83
II.K.3.31	Plant Specific Calculations	12/21/83
II.K.3.44	Evaluation of Anticipated Transients Combined with Single Failure	6/18/82
II.K.3.45	Depressurization with systems other than ADS	5/17/83
III.A.1.1	Emergency Preparedness-Short Term	5/8/80
III.A.2.1	Emergency Preparedness-Upgrade Emergency Plans	5/9/83
III.D.1.1	Primary Coolant Outside Containment Leak Reduction Program	5/8/80
III.D.3.3	Inplant Radiation Monitoring	5/8/80

* These programs are implemented by the NRC.

Table 2

THE FOLLOWING IS A STATUS OF OUR REMAINING OPEN ITEMS:

I.C.1 Guidance for the Evaluation and Development of Procedures for Transients and Accidents (EOPs)

Status: As stated in our most recent correspondence dated April 15, 1983, our EOPs will be implemented in two phases. The phase 1 EOPs were in place by November, 1983. These did not include ATWS events. Phase 2 will be implemented prior to the restart from our Cycle 11 refueling outage and will include all of the items omitted in phase 1.

I.D.1 Control Room Design Reviews (CRDR)

Status: The CRDR for Oyster Creek was essentially completed in September 1982. Further integration of EOPs are necessary. A summary report of the completed CRDR for all items required by Generic Letter 82-33 was submitted to the NRC by letter dated April 30, 1984.

I.D.2 Safety Parameter Display System

Status: A formal Safety Evaluation was completed in April 1984. The implementation plan will be submitted in June 1984, and the SPDS is projected to be operational prior to restart from our Cycle 11 outage.

II.B.1 Reactor Coolant System Vents

Status: By letter dated September 2, 1983, the NRC requested additional information including our detailed isolation condenser high point vent modification design. We plan to provide this information by December 1, 1984.

II.B.2 Plant Shielding

Status: This item is complete with the exception of change out of filters for the Standby Gas Treatment System. By letter dated April 15, 1983, GPUN requested cancellation of this requirement based on NRC staff evaluation of SEP Topic XV-19. The results of that analyses demonstrated that a single STBGS filter train was capable of handling an entire accident. Additional information regarding the adequacy of our plant shielding design review has been requested by the NRC.

II.B.3 Post Accident Sampling

Status: By correspondence dated February 10, 1984 we provided the following dates for the implementation of our PASS System:

- 1) Completion of Hardware Installation - Mid August
- 2) Completion of Startup and Test - Mid September
- 3) Turnover to Plant - Mid September/End of October

II.D.3 Valve Position Indication

Status: Positive indication of Relief and Safety Valve Position has been installed and the Oyster Creek Technical Specifications reflect their operability. Environmental qualification will be accomplished in accordance with 10 CFR 50.49.

II.E.4.1 Dedicated Hydrogen Penetrations

Status: By letter dated January 17, 1983, the NRC stated that the requirements of 10 CFR 50.44 are appropriate in order to conform to this action item.

II.F.1.1 Noble Gas Monitor

Status: This is scheduled to be completed during our present refueling outage.

II.F.1.2 Iodine/Particulate Sampling

Status: This is scheduled to be completed during our present refueling outage.

II.F.1.3 Containment High Range Monitor

Status: This will be installed during our Cycle 11 refueling outage.

II.F.2 Instrumentation for Detection of Inadequate Core Cooling

Status: By letter dated April 4, 1980 and February 10, 1981, we stated "... the existing safety grade multiple water level instrumentation is a direct and unambiguous indication of inadequate core cooling." We are still in agreement with this position and no new modifications are planned.

II.K.3.18 ADS Logic

Status: A plant specific risk analysis has been completed. No ADS Logic Modifications are recommended for Oyster Creek.

II.K.3.19 Interlock for Recirc Pump Modification

Status: By NRC correspondence dated March 14, 1983 the NRC has confirmed our deferral of this modification to Cycle 11.

II.K.3.27 Common Reference Level

Status: By GPUN correspondence dated April 8, 1983. The final phase of implementation for this modification which involves the deletion of all references to old scales, procedure revisions, and conducting the necessary training, will be completed during our current refueling outage.

II.K.3.46 Michelson Concerns

Status: By GPUN correspondence dated March 10, 1982, we stated that the design of the new overhead grid core spray sparger system has taken water hammer effects into consideration. We are awaiting further NRC evaluation of this topic.

III.A.1.2 Upgrade Emergency Support Facilities

Status: As discussed in our response to 0737 Supplement 1, our OSC is operational. The new TSC building should be completed in June 1984, and operational (without Data System) in August 1984. Our EOF has been relocated to our former FEOF in Lakewood, NJ.

III.A.2.2 Meteorological Collection System

Status: The Met Tower at Oyster Creek has no independent power supply as yet. Further study is on-going in this area.

III.D.3.4 Control Room Habitability

Status: As stated in correspondence dated April 8, 1983 interim modifications are scheduled for completion during our Cycle 11 refueling outage. Final modifications will be completed at a later date. The NRC has requested further information on this item.