

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)

EDWIN I. HATCH, UNIT 1

DOCKET NUMBER (2)

0 5 0 0 0 3 2 1 1 OF 0 2

PAGE (3)

TITLE (4)

UNPLANNED REACTOR SCRAM

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)									
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)							
0	6	0	8	4	0	0	6	2	9	8	4	0	5	0	0	0		
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)															
1			20.402(b)			20.405(c)			<input checked="" type="checkbox"/> 50.73(a)(2)(iv)			73.71(b)						
POWER LEVEL (10)			20.405(a)(1)(i)			50.36(c)(1)			50.73(a)(2)(v)			73.71(c)						
Q 0 8			20.405(a)(1)(ii)			50.36(c)(2)			50.73(a)(2)(vii)			OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
			20.405(a)(1)(iii)			50.73(a)(2)(i)			50.73(a)(2)(viii)(A)									
			20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)									
			20.405(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(ix)									

LICENSEE CONTACT FOR THIS LER (12)

NAME

T. L. Elton, Acting Superintendent of Regulatory Compliance

TELEPHONE NUMBER

AREA CODE

9 1 2 3 6 7 1 7 8 5 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 03-11-84 operating personnel were reducing reactor power by inserting control rods. During the process of inserting control rods, it was determined that control rod 50-31 was three position notches away from the position of control rod 02-23.

"CONTROL ROD MOVEMENT" procedure (HNP-1-9207) requires that the reactor be manually scrammed if a control rod within a group skips more than one notch when the reactor is at less than 20% power (the reactor was at approximately 8% power). The manual scram was not a preplanned event; therefore, it is reportable per 10CFR 50.73(a)(2)(iv).

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)  EDWIN I. HATCH, UNIT 1	DOCKET NUMBER (2)  0 5 0 0 0 3 2 1 8 4 — 0 0 6 — 0 0 0 2 OF 0 2	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			

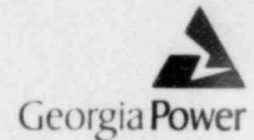
TEXT (If more space is required, use additional NRC Form 368A's) (17)

This 30 day LER is required by 10CFR 50.73(a)(2)(iv) because the reactor manual scram which occurred on 03-11-84 was not part of a preplanned sequence during testing or reactor operation.

On 03-11-84, with the reactor in descending power at 194 MWT (approximately 8% power), operating personnel were reducing reactor power by inserting control rods. During the control rod insertion process per the "CONTROL ROD MOVEMENT" procedure (HNP-1-9207), it was determined that control rod 50-31 was three (3) position notches away from the position of control rod 02-23 (both control rods were within group 19). The reactor was then manually scrammed per HNP-1-9207, which requires a manual scram if a control rod in a group of control rods skips more than one notch when the reactor is operating at less than 20% power. The unplanned reactor scram placed the event within the reporting requirements of 10CFR 50.73(a)(2)(iv). The event occurred on 03-11-84; at the time it was determined that the event was not significant enough to justify a Deficiency Report per the "DEFICIENCY CONTROL SYSTEM" procedure (HNP-444). However, on 06-04-84 during an investigation of a different event, the Regulatory Compliance department found that the scram event had not been reported. After further discussions, a Deficiency Report was initiated on 06-07-84.

The reactor vessel experienced no pressure transients during this event. The vessel water level was controlled with the reactor feed pump. There were no actual or potential safety consequences of this event. All redundant and backup systems remained operable during this event. This event had no impact upon any other system in Unit 1, or Unit 2. This event is non-repetitive.

Georgia Power Company  
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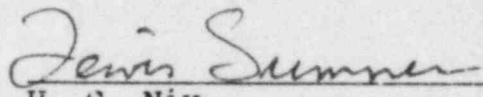
Edwin I. Hatch Nuclear Plant

June 29, 1984  
GM-84-549

PLANT E. I. HATCH  
Licensee Event Report  
Docket No. 50-321

United States Nuclear Regulatory Commission  
Document Control Desk  
Washington, D. C. 20555

Attached is Licensee Event Report No. 50-321/1984-006. This report is required by 10CFR 50.73(a)(2)(iv).

  
for H. C. Nix  
General Manager

*JEE*  
HCN/TLE/vlt

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IE-22  
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