

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Quad-Cities Nuclear Power Station, Unit One										DOCKET NUMBER (2) 0 5 0 0 0 2 5 4				PAGE (3) 1 OF 0 2		
TITLE (4) Unit One Reactor Scram																
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)			
0 5	0 3	8 4	8 4	0 1 0	0 0	0 6	2 5	8 4					0 5 0 0 0			
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11)														
POWER LEVEL (10)		20.402(b)				20.406(c)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)				73.71(b)		
		20.406(a)(1)(i)				50.36(c)(1)				<input type="checkbox"/> 50.73(a)(2)(v)				73.71(c)		
		20.406(a)(1)(ii)				50.36(c)(2)				<input type="checkbox"/> 50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)		
		20.406(a)(1)(iii)				50.73(a)(2)(i)				<input type="checkbox"/> 50.73(a)(2)(viii)(A)						
		20.406(a)(1)(iv)				50.73(a)(2)(ii)				<input type="checkbox"/> 50.73(a)(2)(viii)(B)						
		20.406(a)(1)(v)				50.73(a)(2)(iii)				<input type="checkbox"/> 50.73(a)(2)(ix)						
LICENSEE CONTACT FOR THIS LER (12)																
NAME Craig Iben										TELEPHONE NUMBER AREA CODE 3 0 9 6 5 4 - 2 2 4 1						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NFRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NFRDS						
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On May 30, 1984, a trip of the Reactor Protection System (RPS) was experienced on Unit One due to IRMs 11, 14, 15 and 17 going Hi-Hi. There was no apparent root cause for the trip. Safety implications due to the trip were minimal as there was no fuel in the vessel at the time of the occurrence.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Quad-Cities Nuclear Power Station, Unit One	0 5 0 0 0 2 5 4 8 4	—	0 1 0	—	0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Event Description

During the refueling outage on May 30, 1984, Unit One was in the STARTUP mode with no fuel in the vessel. At 9:17 a.m. the unit scrambled from an IRM Hi-Hi spike on IRMs 11, 14, 15, and 17. The scram relays were reset immediately. Approximately one minute later, IRMs 11, 14, 15, and 17 again spiked Hi-Hi with a resultant scram. The scram relays were reset once again. All other systems remained normal. There were no adverse safety consequences attributed to this deviation since there was no fuel in the vessel at the time of the occurrence.

Cause

There is no apparent root cause for this deviation. It can only be surmised that personnel working near the instruments may have inadvertently caused the spiking by bumping the instruments or welding near the instruments. No event of this type has occurred previously, nor has the event re-occurred since.

Corrective Action

The immediate corrective action was to reset the scram relays for both scrams. Instrument Maintenance performed the IRM functional surveillance that same day with no problems encountered. Since the event was considered spurious, no further corrective action was deemed necessary.



**Commonwealth Edison**

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Cordova, Illinois 61242  
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NJK-84-204

June 25, 1984

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

Reference: Quad-Cities Nuclear Power Station  
Docket Number 50-254, DPR-29, Unit One

Enclosed please find Licensee Event Report Number (LER) 84-10  
for Quad-Cities Nuclear Power Station.

This report is submitted to you in accordance with the requirements of the Code of Federal Regulations, Title 10, Part 50.73(a)(2)-(iv), which is concerned with any event or condition that resulted in manual or automatic actuation of any Engineered Safety Feature.

Respectfully,

COMMONWEALTH EDISON COMPANY  
QUAD-CITIES NUCLEAR POWER STATION

N. J. Kalivisnakis  
Station Superintendent

NJK:DBC/bb

Enclosure

cc B. Rybak  
A. Morrongiello  
INPO Records Center  
NRC Region III  
ANI Library

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