

APPROVED BY OMB  
3150-0011  
EXPIRES 4-30-82

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	S	C	V	C	S	1	2	0	0	-	0	0	0	0	0	-	0	0	3	4	1	0	0	0	4			5		
7	8	9					14		15											25		26					30		37	CAT	58
LICENSEE CODE								LICENSEE NUMBER												LICENSE TYPE											

CON'T

01 REPORT SOURCE L 6 0 5 0 0 0 3 9 5 7 1 1 1 2 3 3 8 0 6 2 9 8 4 9  
7 8 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

## EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 During a planned shutdown, Engineered Safety Features (ESF) 120 VAC vital instrumentation panel APN-5901 was transferred to alternate power to

0 3

0 4 accommodate modifications to its normal power source. With Train "A" Residual Heat Removal System in service, its suction valve XVG-8701A closed. The valve

0 5

0 6 was reopened within approximately five minutes. No adverse consequences resulted

0 7

0 8 due to plant conditions and the short duration of the event. The plant remained in compliance with Technical Specifications during the event.

SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE				COMP. SUBCODE		VALVE SUBCODE	
0	9	C	B	D	Z	I	N	S	T	R	U	P	Z
7	8	9	10	11	12	13	14	15	16	17	18	19	20
(17) LER/RO REPORT NUMBER		EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.			
8		3		1		3		L		1			
21		22		23		24		25		26			
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED			
X	X	Z	Z	0	0	0	0	Y	N	N	W		
33	34	35	36	37	38	39	40	41	42	43	44		

## CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 A dead bus transfer from normal to alternate power source created a power  
1 1 transient in the associated ESF instrumentation bus. Erroneous signals were  
1 2 generated as a result of the transient. Conditions were returned to normal  
1 3 after the transfer was completed. A power distribution list is to be generated  
1 4 to inform operators of plant instrumentation power sources.

0 9  
FACILITY STATUS      % POWER      OTHER STATUS (30)      METHOD OF DISCOVERY      DISCOVERY DESCRIPTION (32)

1 5 D (28) 0 0 0 (29) N/A A (31) Operator Observation

2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20

ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)

1 6 Z (33) Z (34) N/A N/A

7 8 9 10 11 44 45 80

PERSONNEL EXPOSURES		NUMBER		TYPE		DESCRIPTION	
1	7	0	0	0	0	0	N/A

PERSONNEL INJURIES  
NUMBER DESCRIPTION (41) N/A

1 8 0 0 0 (40)

8407030169 840629  
RDR ADPOCK 05000325

LOSS OF OR DAMAGE TO FACILITY  
TYPE DESCRIPTION (43) N/A

2		0		N		45		N/A		NRC USE ONLY																	
7	6	5	4	3	2	1	0	9	8	7	6	5	4	3	2	1	0	9	8	7	6	5	4	3	2	1	0

NAME OF PREPARER

PHONE:

C. J. McKinney

(803) 345-5209

#### EVENT DESCRIPTION AND PROBABLE CONSEQUENCES

On November 28, 1983, the Plant was shutdown for post-steam generator modification eddy current testing. The Plant was in Mode 5 with Train "A" Residual Heat Removal (RHR) System in service.

A modification was scheduled to be performed on Engineered Safety Features (ESF) inverter, XIT-5901, while the Plant was shut down. XIT-5901 is the normal power source for APN-5901, a 120 VAC vital instrument power supply panel for ESF instrumentation. In order to perform the modification on the inverter, APN-5901 was transferred to its alternate power supply, APN-1FA.

Approximately five minutes had elapsed when the Operator at the Controls (OATC) noticed RCS temperature increasing. Upon checking the RHR valve line-up, the OATC discovered XVG-8701A, "RHR Suction Isolation Valve," from the RCS closed. The valve was immediately reopened to restore Train "A" RHR to service. RCS temperature reached 124°F maximum. XVG-8701A was reopened within approximately five minutes of its closure.

Due to plant conditions and the short duration of the event, there were no resulting adverse consequences. The loss of the Train "A" RHR System necessitated entry into Action Statement (b) of Technical Specification 3.4.1.4.2. Technical Specification compliance was maintained at all times.

#### CAUSE AND CORRECTIVE ACTIONS

Pressure transmitter, PT-403, receives loop power from APN-5901 via instrument power panel XPN-7001. PT-403 is the pressure instrument for XVG-8701A. In transferring APN-5901 from normal to alternate power supply, the ESF instrument bus powered by the power panel was momentarily de-energized which created a dead bus transfer. The power transient resulted in the spiking of PT-403 and subsequent closure of XVG-8701A.

This condition returned to normal operation upon completion of the electrical line-up.

An APN power distribution list will be generated by December 31, 1984, to enhance operator anticipation of plant instrumentation responses during transients of this type which may occur in the future.

SOUTH CAROLINA ELECTRIC & GAS COMPANY

POST OFFICE 764

COLUMBIA, SOUTH CAROLINA 29218

O. W. DIXON, JR.  
VICE PRESIDENT  
NUCLEAR OPERATIONS

June 29, 1984

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

SUBJECT: Virgil C. Summer Nuclear Station  
Docket No. 50/395  
Operating License No. NPF-12  
LER 83-136, Revision 1

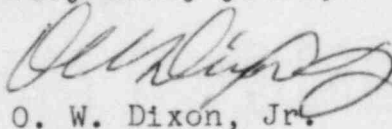
Dear Sir:

Please find attached Revision 1 to Licensee Event Report #83-136 for the Virgil C. Summer Nuclear Station. This Thirty Day Report was previously submitted on December 27, 1983, in accordance with the requirements of Technical Specification 6.9.1.13.(b) and reported an entry into Action Statement (b) of Technical Specification 3.4.1.4.2.

This Revision provides an updated completion schedule for an APN power distribution list. The Licensee has initiated the development of the list; however, a comprehensive list will not be completed until December 31, 1984. The completion of this commitment has been delayed because of the complexity of the task and manpower limitations.

Should there be any questions, please call us at your convenience.

Very truly yours,



O. W. Dixon, Jr.

CJM:OWD/dwf  
Attachment

cc: V. C. Summer  
T. C. Nichols, Jr./O. W. Dixon, Jr.  
E. H. Crews, Jr.  
E. C. Roberts  
W. A. Williams, Jr.  
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Group Managers  
O. S. Bradham  
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