

LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

MAY 1984

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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I. INTRODUCTION

The LaSalle Nuclear Power Station is a Two Unit Facility Located in Marseilles, Illinois. Each Unit is a Boiling Water Reactor with a designed electrical output of 1078 MWe net. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. Unit One is subject to License Number NPF-11, issued on April 17, 1982. The date of initial criticality was June 21, 1982. The unit commenced commercial generation of power on January 1, 1984. Unit Two is subject to license number NPF-18, issued on December 16, 1983. The date of initial criticality was March 10, 1984. The Unit is expected to commence commercial generation of power in August, '84.

This report was compiled by Randy S. Dus telephone number (815)357-6761, extension 324.

II. MONTHLY REPORT FOR UNIT ONE

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

May 1-31 The Unit started the reporting period at 97% power with the main generator on line. At 0100 hours on May 6, reactor power was reduced to 68% for a rod sequence change. At 0000 hours on May 8, reactor power was raised to 81%. At 0730 hours on May 9, reactor power was raised to 95%. At 2220 hours on May 13, commenced load drop to 94% from 98% for turbine bypass valve testing. At 1430 hours on May 14, reactor power was again raised to 98%. At 0055 hours on May 27, reactor power was reduced to 46% due to a TDRFP trip. At 1500 hours on May 27, reactor power was raised to 63%. At 0000 hours on May 28, reactor power was raised to 75%. At 1500 hours on May 28, reactor power was raised to 86%. At 0700 hours on May 29, reactor power was raised to 93%. At 1620 hours on May 31, the main turbine tripped due to a loss of condenser vacuum and a reactor scram followed. The reactor was critical for 736 hours and 20 minutes.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED
MAINTENANCE.

1. Amendments to facility license or Technical Specification.

There were no amendments to the facility license or Technical Specification.

2. Facility or procedure changes requiring NRC approval.

There were no facility or procedure changes requiring NRC approval.

3. Tests and Experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval.

4. Corrective maintenance of safety related equipment.

The following table (Table 1) presents a summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request numbers, LER numbers, Component Name, Cause of Malfunction, Results and Effects on Safe Operation, and Corrective Action.

TABLE 1
CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

LTP-300-7
Revision 3
March 1, 1983
5

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L22394		RHR Press. control VLV	Would not pass adequate supply of steam to Ht.	Increase steam condensing time	Modified press. control valve.
L24416		Drywell shield wall doors	Not installed, leaving openings in sacrificial shield.	Additional heat input to drywell.	Installed shield wall doors.
L29821		Diesel Gen- erator con- trol circuit	Undersized wiring causing excess burden on current trans.	D/G trip with concurrent loss of offsite power and ECCS initiation.	Replaced temporary wiring with perma- nent fix.
L35404		Cable Tray	Debris found in cable tray.	Potential Fire hazard	Cleaned cable tray.
L35815		SBGT Rad Monitor	Particulate channel does not respond to source.	Unknown particulate count	Replaced defective components and recalibrated.
L36671		Diesel Gen- erator	Low oil level in D/G Governor Reservoir.	Potential loss of speed control.	Fill oil reservoir.
L37111		ADS Valve pressure switch.	Low Accumulator pressure alarm up	Adequate pressure still available.	Recalibrated pressure switch.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, occurring during the reporting period, May 1 through May 31, 1984. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
84-022-00	4/14/84	Reactor Scram on Low RPV level
84-023-00	4/15/84	Reactor Water Clean up System PCIS Isolation on High Differential Flow.
84-024-00	5/2/84	Electrical Cable Penetrations Inoperable.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LaSalle One
DATE June 5, 1984
COMPLETED BY Randy S. Dus
TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: May 1984 GROSS HOURS IN REPORTING PERIOD: 744
 2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
 4. REASONS FOR RESTRICTION (IF ANY):
- | | THIS MONTH | YR TO DATE | CUMULATIVE |
|---|----------------|----------------|----------------|
| 5. NUMBER OF HOURS REACTOR WAS CRITICAL | <u>736.3</u> | <u>2594.4</u> | <u>2594.4</u> |
| 6. REACTOR RESERVE SHUTDOWN HOURS | <u>7.7</u> | <u>1019.7</u> | <u>1019.7</u> |
| 7. HOURS GENERATOR ON LINE | <u>736.3</u> | <u>2456.6</u> | <u>2456.6</u> |
| 8. UNIT RESERVE SHUTDOWN HOURS | <u>0.0</u> | <u>1.0</u> | <u>1.0</u> |
| 9. GROSS THERMAL ENERGY GENERATED (MWH) | <u>2120699</u> | <u>6767940</u> | <u>6767940</u> |
| 10. GROSS ELEC. ENERGY GENERATED (MWH) | <u>777907</u> | <u>2229872</u> | <u>2229872</u> |
| 11. NET ELEC. ENERGY GENERATED (MWH) | <u>750769</u> | <u>2119305</u> | <u>2119305</u> |
| 12. REACTOR SERVICE FACTOR | <u>99.0%</u> | <u>71.1%</u> | <u>71.6%</u> |
| 13. REACTOR AVAILABILITY FACTOR | <u>100%</u> | <u>99.1%</u> | <u>99.8%</u> |
| 14. UNIT SERVICE FACTOR | <u>99.0%</u> | <u>67.3%</u> | <u>67.8%</u> |
| 15. UNIT AVAILABILITY FACTOR | <u>99.0%</u> | <u>67.3%</u> | <u>67.8%</u> |
| 16. UNIT CAPACITY FACTOR (USING MDC) | <u>97.4%</u> | <u>56.1%</u> | <u>56.5%</u> |
| 17. UNIT CAPACITY FACTOR (USING DESIGN MWe) | <u>93.6%</u> | <u>53.9%</u> | <u>54.3%</u> |
| 18. UNIT FORCED OUTAGE RATE | <u>1.0%</u> | <u>27.4%</u> | <u>27.4%</u> |
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)
On October 1, 1984 there will be a four week outage to inspect the drywell and perform scheduled surveillances.
 20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: June 3, 1984
 21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):
- | | FORECAST | ACHIEVED |
|----------------------|-------------------|----------------|
| INITIAL CRITICALITY | <u> </u> | <u>6/21/82</u> |
| INITIAL ELECTRICITY | <u> </u> | <u>9/04/82</u> |
| COMMERCIAL OPERATION | <u> </u> | <u>1/1/84</u> |

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-373
UNIT: LASALLE ONE
DATE: JUNE 5, 1984
COMPLETED BY: Randy S. Dus
TELEPHONE: (815) 357-6761

MONTH: May 1984

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

AVERAGE DAILY POWER LEVEL
(MWe-Net)

1. 1083	1. 1081
2. 1085	18. 1089
3. 1076	19. 1079
4. 1073	20. 976
5. 1059	21. 1058
6. 669	22. 1065
7. 745	23. 1067
8. 941	24. 1065
9. 1064	25. 1068
10. 1085	26. 1065
11. 1090	27. 588
12. 1068	28. 886
13. 1081	29. 1031
14. 1065	30. 1080
15. 1089	31. 721
16. 1093	

INSTRUCTIONS

On this form list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line.) In such cases the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

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Revision 3
March 1, 1983
9 (Final)

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH MAY 1984

DOCKET NO. 050-374
UNIT NAME LaSalle One
DATE June 5, 1984
COMPLETED BY Randy S. Dus
TELEPHONE (815)357-6761

NO.	DATE	TYPE	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED S: SCHEDULED				
9	5/31/84	F	7.7	A	3	Loss of condenser vacuum as a result of blown SJAE & S.P.E. Loop Seals. Resulted in Turbine trip & Reactor scram. Procedure revisions in progress to ensure loop seals remain filled.

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief valve operations for Unit One.

There were no relief valve operations for Unit One for this reporting period.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
1-430-84	1A D/G	Lubrication
1-438-84	LPCS W/L Pump	Repair Oil Bubbler
1-442-84	WPCS Pump Breaker	Inspect Breaker Switch.

3. Off-Site Dose Calculation Manual

There were no changes to the off-site dose calculations manual during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no changes to the Radioactive Waste Treatment System during this reporting period.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

MAY 1984

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

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 - 1. Safety/Relief Valve Operations
 - 2. ECCS System Outages
 - 3. Off-Site Dose Calculation Manual Changes
 - 4. Major Changes to Radioactive Waste Treatment System

I. INTRODUCTION

The LaSalle Nuclear Power Station is a Two Unit Facility Located in Marseilles, Illinois. Each Unit is a Boiling Water Reactor with a designed electrical output of 1078 MWe net. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. Unit One is subject to License Number NPF-11, issued on April 17, 1982. The date of initial criticality was June 21, 1982. The unit commenced commercial generation of power on January 1, 1984. Unit Two is subject to license number NPF-18, issued on December 16, 1983. The date of initial criticality was March 10, 1984. The Unit is expected to commence commercial generation of power in August, '84.

This report was compiled by Randy S. Dus, telephone number (815)357-6761, extension 324.

II. MONTHLY REPORT FOR UNIT TWO

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT TWO

May 1-5 The unit started the reporting period at 21% power with the main generator on line. At 0500 hours on May 2, the main turbine was tripped during surveillance LOS-TG-W1. At 0515 hours on May 2, the main turbine was back on line. At 1030 hours on May 3 the turbine was again tripped during startup testing. At 2135 hours on May 3, commenced normal unit shutdown to hot standby for maintenance work. At 2340 hours on May 3, the reactor was manually scrammed due to reactor water level transients. The reactor was critical for 71 hours and 40 minutes.

May 6-21 The reactor went critical at 1700 hours on May 6. At 0506 hours on May 8 the main generator was synchronized to the grid. At 0700 hours on May 8, reactor power was raised to 21%. At 2330 hours on May 11, commenced normal reactor shutdown for turbine maintenance. At 0102 hours on May 12, the main turbine was manually tripped. At 1640 hours on May 12, the reactor was placed back in "run" mode. At 1723 hours on May 12, the main generator was synchronized to the grid. At 2300 hours on May 12, reactor power was raised to 24%. At 1545 hours on May 17, the main turbine was tripped for STP 27-2. At 1625 hours on May 17, the main generator was synchronized to the grid. At 0200 hours on May 19, reactor power was raised to 34%. At 1825 hours on May 21, the main turbine tripped with a subsequent reactor scram. The reactor was critical for 361 hours and 25 minutes.

May 22-24 The reactor went critical at 2258 hours on May 22. At 2315 hours on May 22 commenced reactor shutdown. At 0301 hours on May 23, the mode switch was taken to "Shutdown". The reactor was critical for 4 hours and 3 minutes.

May 25-31 The reactor went critical at 1625 hours on May 25. At 1900 hours on May 26, the main generator was synchronized to the grid. At 1954 hours on May 26, the main turbine tripped on high level in the MSR. At 2026 hours on May 26 the main turbine was synchronized to the grid. At 0610 hours on May 27, commenced reactor shutdown to repair the number 3 bypass valve. At 0738 hours on May 27, the main turbine was tripped. At 1500 hours on May 27, commenced startup from hot standby to power operation. At 2240 hours on May 27, the reactor mode switch was taken to run and the reactor power level was 6%. At 0055 hours on May 28, the main generator was synchronized to the grid. At 0700 hours on May 28, reactor power was raised to 24%. At 2300 hours on May 28, reactor power was raised to 44%. At 1500 hours on May 29, reactor power was at 25%. The Unit completed the reporting period at 26% power. The reactor was critical for 151 hours and 35 minutes.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specifications.

There were no amendments to the facility license or Technical Specifications during the reporting period.

2. Facility or procedure changes requiring NRC approval.

There were no facility or procedure changes requiring NRC approval during the reporting period.

3. Tests and experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval during the reporting period.

4. Corrective Maintenance of Safety Related Equipment.

The following table (Table 1) presents a summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request numbers, LER Numbers, Component Name, cause of malfunction, results and effects on safe operation, and corrective action.

TABLE 1
CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

LTP-300-7
Revision 3
March 1, 1983
5

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L35686		Mainsteam line D/P Gauge	Indicating needle bent	Inaccurate MSL OPI reading	Straightened needle and recalibrated.
L36018		Rx Bldg B PRM	PRM is reading decade above other detectors	None. Redundant channels still operable.	Replaced GM Tube and recalibrated.
L36066		Ammonia detector	Broken power lead to alarm relay.	Causes spurious alarm	Repaired faulty lead
L36367		Rx Bldg to Aux Bldg. Air lock	Both doors open at same time	Potential loss of secondary containment	Repaired interlock
L36438		Div I Post Loca monitor	Instrumentation Drift	Unknown H ₂ & O ₂ concen- trations during Loca	Recalibrated post Loca monitor.
L36504		Drywell Airlock	Interlock mechanism sticks	Door sticks, potential personnel safety.	Adjusted Linkage
L36888	84-018-00	HPCS Pump brkr.	HPCS Breaker did not close on attempt to start pump.	Unable to start HPCS pump	Replaced faulty con- tacts.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit Two, occurring during the reporting period, May 1 through May 31, 1984. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
84-012-00	4/26/84	Reactor Manual scram due to loss of normal feedwater.
34-013-00	4/3/84	Reactor Water clean up Differential pressure isolation.
84-014-00	4/3/84	High Pressure core spray jockey pump failure.
84-015-00	4/11/84	Failure to realize limiting condition of operation prior to changing mode.
84-016-00	4/23/84	Reactor water clean up high ambient temperature isolation.
84-017-00	5/3/84	Reactor scram on loss of feedwater.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-374
UNIT LaSalle Two
DATE June 5, 1984
COMPLETED BY Aras R. Lintakas
TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: May 1984 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3323 MAX DEPEND CAPACITY
(MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
4. REASONS FOR RESTRICTION (IF ANY):
5. NUMBER OF HOURS REACTOR WAS CRITICAL THIS MONTH YR TO DATE CUMULATIVE
6. REACTOR RESERVE SHUT DOWN HOURS 588.7 1296.8 1296.8
7. HOURS GENERATOR ON LINE 155.3 686.0 686.0
8. UNIT RESERVE SHUTDOWN HOURS 473.7 581.1 581.1
9. GROSS THERMAL ENERGY GENERATED (MWH) 0.0 0.0 0.0
10. GROSS ELEC. ENERGY GENERATED (MWH) 444715 596465 596465
11. NET ELEC. ENERGY GENERATED (MWH) 110201 126884 126884
12. REACTOR SERVICE FACTOR 98877 113093 113093
13. REACTOR AVAILABILITY FACTOR N/A N/A N/A
14. UNIT SERVICE FACTOR N/A N/A N/A
15. UNIT AVAILABILITY FACTOR N/A N/A N/A
16. UNIT CAPACITY FACTOR (USING MDC) N/A N/A N/A
17. UNIT CAPACITY FACTOR(USING DESIGN MWe) N/A N/A N/A
18. UNIT FORCED OUTAGE RATE N/A N/A N/A
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: N/A
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):
INITIAL CRITICALITY FORECAST ACHIEVED
INITIAL ELECTRICITY 3/10/84
COMMERCIAL OPERATION 4/20/84
Aug. 84

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-374
UNIT: LASALLE TWO
DATE: June 5, 1984
COMPLETED BY: Randy S. Dus
TELEPHONE: (815) 357-6761
MONTH: May 1984

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	166
2.	171
3.	76
4.	0
5.	0
6.	0
7.	0
8.	135
9.	152
10.	169
11.	168
12.	48
13.	176
14.	176
15.	182
16.	239

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	212
18.	205
19.	302
20.	295
21.	236
22.	0
23.	0
24.	0
25.	0
26.	23
27.	35
28.	291
29.	292
30.	195
31.	178

INSTRUCTIONS

On this form list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line.) In such cases the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

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ATTACHMENT E

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH MAY 1984

DOCKET NO. 050-374
UNIT NAME LaSalle Two
DATE June 5, 1984
COMPLETED BY Randy S. Dus
TELEPHONE (815)357-6761

NO.	DATE	TYPE	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED S: SCHEDULED				
9	5/2/84	S	.3	B	9	Turbine Trip during Startup Testing
10	5/3/84	S	114.6	B	2	Performed Minor Maintenance work
11	5/12/84	S	16.4	B	9	Turbine trip during Startup testing
12	5/17/84	S	.7	B	9	Turbine Trip during Startup testing
13	5/21/84	F	120.6	A	3	Reactor Scram Following Turbine trip from main transformer differential current relay trip.

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March 1, 1983
9 (Final)

ATTACHMENT E

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH MAY 1984

DOCKET NO. 050-374
UNIT NAME LaSalle Two
DATE June 5, 1984
COMPLETED BY Randy S. Dus
TELEPHONE (815)357-6761

NO.	DATE	TYPE	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED S: SCHEDULED				
14	5/26/84	F	.5	A	9	Turbine trip on MSR high level
15	5/27/84	S	17.3	B	9	Turbine trip to perform maintenance on #3 bypass valve.

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief Valve Operations for Unit Two.

There were no relief valve operations for Unit Two for this reporting period.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
2-649-84	Bus 243	HPCS Pump Breaker Testing
2-655-84	South ADS Bottle Bank Pressure Indicator	Repair Leak
2-688-84	HPCS W/L Pump	Lubricate Coupling
2-689-84	LPCS W/L Pump	Lubricate Coupling

3. Off-Site Dose Calculation Manual

There were no changes to the off-site dose calculations manual during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no changes to the Radioactive Waste Treatment System during this reporting period.



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

June 5, 1984

Director, Office of Management Information
and Program Control
United States Nuclear Regulatory Commission
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering
LaSalle County Nuclear Power Station for the period covering May 1 through May
31, 1984.

Very truly yours,

G. J. Diederich
Superintendent
LaSalle County Station

GJD/RSD/crh

Enclosure

xc: J. G. Keppler, NRC, Region III
NRC Resident Inspector LaSalle
Gary Wright, Ill. Dept. of Nuclear Safety
D. P. Galle, CECO
D. L. Ferrar, CECO
INPO Records Center
Ron A. Johnson, PIP Coordinator SNED
W. R. Jackson, GE Resident

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