



NIAGARA MOHAWK POWER CORPORATION/300 ERIE BOULEVARD WEST, SYRACUSE, N.Y. 13202/TELEPHONE (315) 474-1511

June 15, 1984
(NMP2L 0083)

Mr. R. W. Starostecki, Director
U.S. Nuclear Regulatory Commission
Region I
Division of Project and Resident Programs
631 Park Avenue
King of Prussia, PA 19406

Re: Nine Mile Point Unit 2
Docket No. 50-410

Dear Mr. Starostecki:

Enclosed is a final report in accordance with 10CFR50.55(e) for the problem concerning feedwater piping spools (55(e)-84-05). This problem was reported in a telephone conversation between T. Loomis (Nine Mile Point Unit 2 Licensing) and S. Collins of your staff on February 1, 1984. An interim report was submitted via our letter dated March 5, 1984.

Very truly yours,

C. V. Mangan
Vice President
Nuclear Engineering & Licensing

TRL:ja
Enclosure
xc: Director of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, DC 20555

R. A. Gramm, Resident Inspector

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NIAGARA MOHAWK POWER CORPORATION
NINE MILE POINT UNIT 2
DOCKET NO. 50-410

Final Report for a Problem
Concerning Feedwater Piping Spools
(55(e)-84-05)

Description of the Problem

During a Stone & Webster Engineering Corporation audit of ITT Grinnell, it was observed that the material used in the fabrication of some feedwater piping spools did not meet all the impact testing requirements of ASME Section III. Certified Material Test Reports for the material in question did not document the test coupon heat treatment duration and temperature as required by ASME Section III, NB-2211. Further inquiry revealed that the heat treatment of the test coupon was in fact not performed. Heat treatment of the test coupons is required to ensure that the coupons reflect the material conditions of the spool piece. Eight feedwater pipe spools were affected.

The material test coupons with the same heat lot numbers as those of the eight feedwater pipe spools have since been heat treated and impact tested. Impact test results of the coupons indicated that seven spools meet the requirements of ASME Section III. The coupon of the remaining spool piece did not meet the impact test lateral expansion criterion of ASME Section III.

Analysis of Safety Implications

A detailed investigation of the spool piece that did not meet lateral expansion criterion, to determine its suitability for use, was not performed. Instead, this spool piece will be replaced with one meeting ASME III requirements. Had the investigation indicated that the spool piece would not have performed its intended design function, a possibility exists that the integrity of the reactor coolant pressure boundary could have been jeopardized. Therefore, if this problem were to have remained uncorrected, it could have adversely affected the safe operation of the plant.

Corrective Action

The spool piece that did not meet ASME III requirements will be replaced by September 14, 1984.

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