

LICENSEE EVENT REPORT

EXHIBIT A

CONTROL BLOCK: 1 (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 NY N M P 1 00 - 000000 - 000 03 4 1 1 1 1 4 5  
7 8 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 31 CAT 38

01  
7 8

REPORT SOURCE: L 0 5 0 0 0 2 2 0 7 1 0 1 9 8 2 8 0 6 2 5 8 4 9  
60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)  
02 See Attached Sheet

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04  
05  
06  
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09 S C 11 A 12 F 13 F I L T E P 14 Z 15 Z 16  
9 10 11 12 13 14 15 16

17 LEAD REPORT NUMBER 18 EVENT YEAR 19 20 21 22 23 24 25 26 27 28 29 30 31 32  
0 2 0 2 4 0 3 L 0

ACTION TAKEN 18 X 19 G 20 Z 21 Z 22 0 0 0 0 23 Y 24 N 25 X 26 C 0 4 5 27  
33 34 35 36 37 38 39 40 41 42 43 44 45

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)  
10 See Attached Sheet

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12  
13  
14

15 H 28 0 0 0 29 N/A 30 31 N/A 32  
3 8 9 FACILITY STATUS 10 POWER 11 OTHER STATUS 12 METHOD OF DISCOVERY 13 DISCOVERY DESCRIPTION 14

16 Z 33 Z 34 N/A 35 N/A 36  
7 8 9 ACTIVITY RELEASED 10 CONTENT 11 AMOUNT OF ACTIVITY 12 LOCATION OF RELEASE 13

17 0 0 0 37 Z 38 N/A 39  
7 8 9 PERSONNEL EXPOSURES 10 NUMBER 11 TYPE 12 DESCRIPTION 13

18 0 0 0 40 N/A 41  
7 8 9 PERSONNEL INJURIES 10 NUMBER 11 DESCRIPTION 12

19 Z 42 N/A 43  
7 8 9 LOSS OF OR DAMAGE TO FACILITY 10 TYPE 11 DESCRIPTION 12

20 N 44 N/A 45  
7 8 9 PUBLICITY 10 ISSUED 11 DESCRIPTION 12

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PDR ADOCK 05000220 PDR  
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NRC USE ONLY

NAME OF PREPARER A R Schwedt

PHONE (315) 349-2611

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EVENT DESCRIPTION AND PROBABLE CONSEQUENCES:

During a refueling outage, a Q.A. review discovered on May 21, 1984 that documentation of a surveillance test performed on the Reactor Building Emergency Ventilation System on October 19, 1982, indicated that, although visual inspection of doors and access openings for adequacy of sealing was documented, no record was made to verify that a test was also conducted using DOP or FREON as a part of the corresponding filter test. This test is required by Technical Specifications 4.4.4.f, which states "Test sealing of gaskets for housing doors downstream of the HEPA filters and charcoal absorbers shall be performed at and in conformance with each test performed for compliance with specification 4.4.4.b and specification 3.4.4.b." Since the integrity of this system was verified through the successful completion of Operations Surveillance Test N1-ST-M8, "Emergency Ventilation System Operability Test" (which verifies acceptable Reactor Building leakage and negative pressure on the Reactor Building), the overall safety consequences arising out of this event were minimal.

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS:

This event resulted from a vagueness in procedure N1-RTP-38, rev. 0 and rev. 1, "Test and Analysis of HEPA and Charcoal Bed Filters," which required that the test results for the housing doors gasket seal be recorded in the "remarks" section of the contractor's form NCS form #4. A test of the #11 and #12 Reactor Building Emergency Ventilation System was performed on May 30, 1984. No leakage was detected and the results were recorded on contractor's data sheet NCS form #4 in accordance with procedure N1-RTP-38, rev. 1. To prevent further occurrences, procedure N1-RTP-38 will be revised by August 31, 1984 to provide a check list which will insure that all required testing and its documentation is complete.

## NIAGARA MOHAWK POWER CORPORATION

NIAGARA  MOHAWK300 ERIE BOULEVARD, WEST  
SYRACUSE, N. Y. 13202

June 25, 1984

United States Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

RE: Docket No. 50-220  
LER 82-24

Gentlemen:

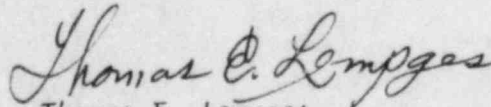
In accordance with Nine Mile Point Nuclear Station Unit #1 Technical Specifications, we hereby submit the following Licensee Event Report:

LER 82-24      Which is being submitted in accordance with Section 6.9.2.b(3), observed inadequacies in the implementation of administrative or procedural controls which threaten to cause reduction of degree of redundancy provided in reactor protection systems or engineered safety systems.

The event occurred in October, 1982, but was not discovered until May 21, 1984 by a Quality Assurance review.

This report was completed in the format designated in NUREG-0161, dated July 1977, in accordance with NUREG-1022, Supp. 1 and discussions with the NRC staff.

Very truly yours,



Thomas E. Lempges  
Vice President  
Nuclear Generation

TEL/lo  
Attachments  
cc: Dr. Thomas E. Murley  
Regional Administrator

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11