TENNESSEE VALLEY AUTHORITY

DIVISION OF NUCLEAR POWER

SEQUOYAH NUCLEAR PLANT

MONTHLY OPERATING REPORT

TO THE

NUCLEAR REGULATORY COMMISSION

MAY 1, 1984 - MAY 31, 1984

UNIT 1

DOCKET NUMBER 50-327

LICENSE NUMBER DPR-77

UNIT 2

DOCKET NUMBER 50-328

LICENSE NUMBER DPR-79

Submitted By: O.R. Wallace

Plant Manager

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Operations Summary

May, 1984

The following summary describes the significant operational activities for the month of May. In support of this summary, a chronological log of significant events is included in this report.

Unit 1

Unit 1 was critical for 259.1 hours, produced 172,230 MWH (gross), resulting in an average hourly gross load of 677,058 kW during the month. There are 362.55 full power days estimated remaining until the end of cycle 3 fuel. With a capacity factor of 85 percent, the target EOC exposure would be reached July 31, 1985. The capacity factor for the month was 19.6 percent.

There were 2 reactor scrams, no manual shutdowns, and no power reductions.

Unit 2

Unit 2 was critical for 728.6 hours, produced 799,390 MWH (gross), resulting in an average hourly gross load of 1,091,583 kW during the month. There are 80.42 full power days estimated remaining until the end of cycle 2 fuel. With a capacity factor of 85 percent, the target EOC exposure would be reached September 3, 1984. The capacity factor for the month was 88.6 percent.

There was 1 reactor scram, no manual shutdowns, and one power reduction during May.

Significant Operational Events

Unit 1

Date	Time	Event
05/01/84	0001	The reactor was in mode 5. The forced outage due to the thimble guide tube leak was continuing.
05/04/84	2040	The reactor entered mode 4.
05/05/84	0526	The reactor entered mode 3.
	2329	The reactor entered mode 4. Reduced pressure to repair the pressurizer relief valves.
05/06/84	0520	The reactor entered mode 5.

Significant Operational Events

Unit 1

(Continued)

Date	Time	Event
05/09/84	0010	The reactor entered mode 4.
	0541	The reactor entered mode 3.
05/10/84	1453	The reactor was taken critical.
	1827	The unit was tied on-line.
	2200	The reactor was at 30% power and holding due to steam generator chemistry.
	2322	The reactor tripped due to a Lo-Lo steam generator level.
05/11/84	0850	The reactor was taken critical.
	1400	The reactor tripped due to a Lo-Lo number 2 steam generator level.
	1707	The reactor was taken critical.
	1903	The unit was tied on-line.
	2200	The reactor was at 30% power, producing 299 MWe and was holding for steam generator chemistry.
05/12/84	0849	Began load reduction for the turbine overspeed trip test.
	1059	Holding at 29% turbine load because the feedwater flow regulating valves were oscillating. Began load ascension to 30%.
	2200	Holding 30% reactor power for the turbine overspeed trip test.
05/13/84	1032	Began load decrease for the turbine overspeed trip test.
	1232	The turbine was taken off-line for the trip test.
	1345	Tied the unit back on-line.
	1400	The reactor was at 30% power and holding for steam generator chemistry.
05/14/84	2255	Began power ascension.
05/15/84	1120	The reactor was at 54% power and holding due to problems with the E.H.C. panel.
	1340	Began power ascension.
	2040	The reactor was at 75% power and was holding for incore/excore probe calibration.

Significant Operational Events

Unit 1

(Continued)

Date	Time	Event
05/18/84	1735	Began power ascension.
	2205	The reactor was at 90% power and was holding to verify the reactor coolant system flow calculations.
05/21/84	1640	Began power ascension.
	1942	The reactor obtained 100% power.
	2234	The reactor tripped due to generator electrical trouble.
05/31/84	1153	The reactor entered mode 2.
	1210	The reactor was taken critical.
	1650	Tied the generator on-line.
	1830	The reactor obtained 30% power, producing 276 MWe.
	2359	The reactor was in mode 1 at 30% power, producing 276 MWe and was holding due to steam generator chemistry.

Unit 2

Date	Time	Event
05/01/84	0001	The reactor was in mode 1 at 100% power, producing 1164 MWe.
05/18/84	2217	Began reducing reactor power to add oil to reactor coolant pump #3.
05/19/84	0730	The reactor was at 30% power, producing 285 MWe.
	1157	The reactor tripped on a Lo-Lo steam generator level after MFPT A tripped when the BOP operator isolated the oil pump isolation valve. MFPT B was already off-line.
05/20/84	0328	The reactor was taken critical.
	1757	Tied the unit on-line.
	1835	The reactor was in mode 1 at 30% power, producing 333 MWe and holding due to steam generator chemistry.

Significant Operational Events

Unit 2

(Continued)

Date	Time	Event
05/22/84	1648	Began power ascension.
	1826	The reactor was at 38% power, producing 405 MWe and was holding for maintenance on LCV-6-106A.
	1948	Began power ascension.
05/23/84	0500	The reactor was holding at 98% power for tests results confirming actual reactor power level.
	1238	Began power ascension.
	1400	The reactor obtained 100% power, producing 1160 MWe.
05/31/84	2359	The reactor was in mode 1 at 100% reactor power, producing 1160 MWe.

PORV'S and Safety Valves Summary

No PORV's or safety valves were challenged during the month.

Licensee Events and Special Reports

The following Licensee Event Reports (LER's) were sent during May 1984, to the Nuclear Regulatory Commission.

LER	DESCRIPTION OF EVENT
1-84023	This event was discovered at 1309C on April 2, 1984, while unit 1 was in mode 5 and unit 2 was in mode 1. While performing surveillance instruction (SI) 92, "Remote Shutdown Monitoring Instrumentation - Pressurizer Pressure Channel Calibrations," the alarm indicating lights on the bistables of the power-operated relief valves (PORV) appeared to be reversed for the correct bistable action. The wiring and operation of the bistables and controller module was investigated and thought to be incorrect. The wiring on the PORVs for both units was modified to this new position. Later, the wiring and bistable operation was reviewed and more thoroughly investigated with additional information and drawings. This additional investigation showed that the PORVs had been modified to an inoperable (reverse from normal) state.

LER	DESCRIPTION OF EVENT
1-84023	(Continued) The PORVs were immediately blocked on the operating unit while both PORVs on both units were rewired and tested to be in the correct wiring configuration and operating correctly. The surveillance instruction has been revised to explain PORV operation and to give details of bistable action. The administrative instruction (AI-25) has been revised to better control wiring changes.
1-84024	This event was discovered at 1330C on April 13, 1984, while unit 1 was in mode 3. Unit 1 entered mode 3 at 0950C on April 12, 1984. A limiting condition for operation (LCO) in the technical specifications was not met and a change from mode 4 (less than 350 degrees F) to mode 3 (greater than 350 degrees F) was made. A level transmitter (LT) for steam generator number one was inoperable. The associated bistables for the level transmitter were already tripped due to another LCO. Personnel failed to realize that more than one LCO applied to this LT. The LCO that was recognized allowed operation until the next required functional test in the applicable mode. Another LCO was overlooked, and it did not permit a mode change.
1-84025	At 1500C on April 15, 1984, sampling of the reactor coolant system (RCS) for boron concentration was initiated. This sampling caused pressurizer level transmitter 1-LT-68-320 to become inoperable due to a modification during the previous outage which routed the sense line from the low side tap of the instrument. This event was detected at 1715C after a change from mode 3 to mode 2 was completed. Two LCOs are involved with this event. LCO 3.3.1.1 is applicable in mode 2, and LCO 3.0.4 is not applicable. After mode 2 was reached, the bistable was tripped at 1748C with the instrument discovered inoperable at 1715C. LCO 3.3.3.7 is for accident monitoring instrumentation and allows seven days to return the instrument operable, but 3.0.4 is applicable (i.e., no mode change with instrument inoperable). LCO 3.3.3.7 is applicable for modes 1, 2, and 3 and the instrument should have been made operable prior to mode change. The cause of this event has been attributed to the failure of the operator to realize LT-68-320 as inoperable.
1-84026	At 2148 on April 17, 1984, unit 1 experienced a reactor trip. Unit 1 was in mode 1 at 30% reactor power just prior to the event. A turbine trip occurred due to failure of a generator stator cooling water pump. Subsequent Lo-Lo level in steam generator number three resulted in a reactor trip from approximately 18% reactor power. Unit 1 stabilized at 547 degrees F following the reactor trip.

LER	DESCRIPTION OF EVENT
1-84027	This LER involves three separate incidents. The first containment ventilation isolation (CVI) occurred at 1642C on April 20, 1984 while unit 1 was in mode 5. The second CVI occurred at 1055C on April 25, 1984 and the third CVI occurred at 1116C on April 25, 1984 while unit 1 was in mode 5. A high radiation alarm was actuated which caused a containment ventilation isolation (CVI) to occur. Investigation revealed that a voltage spike occurred as a result of electromagnetic interference (EMI) which was generated by slippage of the filter paper in two incidents and stray signals in another incident. Radiation levels were not above normal during this time. The inadvertent high radiation alarm was reset and the monitor was returned to service. A time delay is being added to the actuation signal to allow time for spikes to decay.
1-84028	The ABI occurred at 2358C on April 17, 1984, while unit 1 was in mode 3. A high radiation alarm was actuated which caused an auxiliary building isolation (ABI) to occur. Investigation revealed that personnel were placing boric acid evaporator 'B' in service and draining the vent header at the same time that the volume control tank was being burped (vented). This simultaneous action increased the vent header pressure and caused excessive gas to be vented causing the auxiliary building ventilation system to isolate.
1-84029	This LER involves three separate incidents. The first auxiliary building isolation (ABI) occurred at 1205C on May 7, 1984, while unit 1 was in mode 5 and unit 2 was in mode 1. The second ABI occurred at 2341C on May 7, 1984, while unit 1 was in mode 5 and unit 2 was in mode 1. The third ABI occurred at 0828C on May 8, 1984, while unit 1 was in mode 5 and unit 2 was in mode 1. A high radiation alarm was actuated which caused an auxiliary building isolation (ABI) to occur. Investigation revealed that in two incidents, because detector output is not stable and the radiation level is so close to the setpoint, normal fluctuations of the detector tripped the alarm. In another incident, the power source for a radiatin monitor was transferred from one board to another which caused an alarm. Radiation levels were not above normal during this time.

Licensee Events and Special Reports

(Continued)

LER	DESCRIPTION OF EVENT
1-84030	On April 19, 1984, unit 1 was in mode 1 (2235 psig, 558 degrees F) at 30% reactor power with maintenance personnel cleaning the incore detector thimble tubes. A high pressure connection on the thimble tube at the seal table failed resulting in a reactor coolant system pressure boundary leak of approximately 25-35 gpm and ejection of one incore detector thimble tube at 2100 CST.

Diesel Generator Failure Reports

There were no diesel generator failure reports transmitted during the month.

Special Reports

There were no special reports transmitted during the month.

Offsite Dose Calculation Manual Changes

There were not any changes to the Sequoyah Nuclear Plant ODCM during the month.

OPERATING DATA REPORT

DOCKET NO. 50-327
DATE JUNE 11,1984
COMPLETED BY M. G. EDDINGS
TELEPHONE (615) 870-6196

OPERATING STATUS

1. UNIT NAME: SEQUOYAH NUCLEAR PLANT, UNIT 1 NOTES:
2. REPORT PERIOD: MAY 1984
3. LICENSED THERMAL POWER(MWT): 3411.0
4. NAMEPLATE RATING (GROSS MWE): 1220.6
5. DESIGN ELECTRICAL RATING (NET MWE): 1148.0
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 1183.0
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 1148.0
8. IF CHANGES OCCUR IN CAPACITY RATINGS(ITEMS NUMBERS 3 THROUGH 7)SINCE LAST REPORT, GIVE REASONS:_____
9. POWER LEVEL TO WHICH RESTRICTED,IF ANY(NET MWE):_____
10. REASONS FOR RESTRICTIONS,IF ANY:_____

	THIS MONTH	YR.-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	744.00	3647.00	25584.00
12. NUMBER OF HOURS REACTOR WAS CRITICAL	259.10	1360.90	15802.46
13. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
14. HOURS GENERATOR ON-LINE	254.40	1254.50	15367.65
15. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
16. GROSS THERMAL ENERGY GENERATED (MWH)	565213.61	3480448.28	48972298.58
17. GROSS ELECTRICAL ENERGY GEN. (MWH)	172230.00	1139920.00	16519056.00
18. NET ELECTRICAL ENERGY GENERATED (MWH)	160030.00	1082181.00	15859109.00
19. UNIT SERVICE FACTOR	34.19	34.40	60.07
20. UNIT AVAILABILITY FACTOR	34.19	34.40	60.07
21. UNIT CAPACITY FACTOR(USING MDC NET)	18.74	25.85	54.00
22. UNIT CAPACITY FACTOR(USING DER NET)	18.74	25.85	54.00
23. UNIT FORCED OUTAGE RATE	65.75	45.11	21.14

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

NOTE THAT THE THE YR.-TO-DATE AND CUMULATIVE VALUES HAVE BEEN UPDATED.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-327
UNIT NAME Sequoyah One
DATE June 4, 1984
COMPLETED BY M. G. Eddings
TELEPHONE (615) 870-6249

REPORT MONTH MAY

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method Of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
7	840419	F	234.5	A	4				Thimble guide tube leak at seal table.
8	840510	F	19.7	A	3				Loop #2 Lo-Lo generator level.
9	840521	S	1.2	B	9				Turbine overspeed trip test.
10	840521	F	234.2	A	3				Generator electrical trouble, first out.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Cont. of Existing Outage
5-Reduction
9-Other

4
Exhibit G-Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

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Exhibit I-Same Source

(9/77)

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-327
UNIT Sequoyah One
DATE June 4, 1984
COMPLETED BY M. Eddings
TELEPHONE (615) 870-6248

MONTH MAY, 1984

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	N/A	17	834.2
2	N/A	18	858.2
3	N/A	19	997.2
4	N/A	20	1002.5
5	N/A	21	997.5
6	N/A	22	N/A
7	N/A	23	N/A
8	N/A	24	N/A
9	N/A	25	N/A
10	239.5	26	N/A
11	244.1	27	N/A
12	263.3	28	N/A
13	221.3	29	N/A
14	249.0	30	N/A
15	533.2	31	63.3
16	801.5		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

OPERATING DATA REPORT

DOCKET NO. 50-328
DATE JUNE 11,1984
COMPLETED BY D.C.DUPREE
TELEPHONE (615)870-6248

OPERATING STATUS

1. UNIT NAME: SEQUOYAH NUCLEAR PLANT, UNIT 2 NOTES:
2. REPORT PERIOD: MAY 1 THRU 31,1984
3. LICENSED THERMAL POWER(MWT): 3411.0
4. NAMEPLATE RATING (GROSS MWE): 1220.6
5. DESIGN ELECTRICAL RATING (NET MWE): 1148.0
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 1183.0
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 1148.0
8. IF CHANGES OCCUR IN CAPACITY RATINGS(ITEMS NUMBERS 3 THROUGH 7)SINCE LAST REPORT, GIVE REASONS:
9. POWER LEVEL TO WHICH RESTRICTED,IF ANY(NET MWE):
10. REASONS FOR RESTRICTIONS,IF ANY:

	THIS MONTH	YR.-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	744.00	3647.00	17544.00
12. NUMBER OF HOURS REACTOR WAS CRITICAL	728.60	3571.40	13932.47
13. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
14. HOURS GENERATOR ON-LINE	714.00	3552.60	13706.92
15. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
16. GROSS THERMAL ENERGY GENERATED (MWH)	2287795.06	11885954.99	44304022.80
17. GROSS ELECTRICAL ENERGY GEN. (MWH)	779390.00	4100910.00	15132850.00
18. NET ELECTRICAL ENERGY GENERATED (MWH)	748394.00	3951797.00	14569534.60
19. UNIT SERVICE FACTOR	95.97	97.41	78.13
20. UNIT AVAILABILITY FACTOR	95.97	97.41	78.13
21. UNIT CAPACITY FACTOR(USING MDC NET)	87.62	94.39	72.34
22. UNIT CAPACITY FACTOR(USING DER NET)	87.62	94.39	72.34
23. UNIT FORCED OUTAGE RATE	4.03	2.59	7.62

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
Refueling/Modification - Cycle 2 Fuel - September 3, 1984, approximately 51 days.

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

NOTE THAT THE THE YR.-TO-DATE AND CUMULATIVE VALUES HAVE BEEN UPDATED.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-328
UNIT NAME Sequoyah Two
DATE June 11, 1984
COMPLETED BY D. C. Dupree
TELEPHONE (615) 870-6248

REPORT MONTH MAY

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method Of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
2	840518	F	0	B	5				Drop load to add oil to #3 R.C.P.
3	840519	F	30.0	G	3				U. O. tripped oil pump on "A" M.F.P.T. while "B" M.F.P.T. was out of service.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Cont. of Existing Outage
5-Reduction
9-Other

4
Exhibit G-Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I-Same Source

(9/77)

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-328
UNIT Sequoyah Two
DATE June 11, 1984
COMPLETED BY D. C. Dupree
TELEPHONE (615) 870-6248

MONTH May, 1984

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	1136.0	17	1121.8
2	1131.0	18	1122.0
3	1130.0	19	269.0
4	1129.0	20	68.0
5	1129.0	21	295.0
6	1126.0	22	360.0
7	1125.0	23	1085.0
8	1126.0	24	1131.0
9	1126.0	25	1131.0
10	1125.0	26	1131.0
11	1122.0	27	1130.0
12	1119.0	28	1133.0
13	1120.0	29	1131.0
14	1119.1	30	1132.0
15	1119.0	31	1131.0
16	1118.0		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

Plant Maintenance Summary

The following significant maintenance items were completed during the month of May 1984:

Mechanical Maintenance

1. Completed work on the D-12 thimble failure in the unit #1 incore instrument room seal table.

2. Replaced the bad oil seal, gear reducer, and motor seals on the unit #1 MFPT "1B" turning gear.

3. Repaired the solenoid valves on 1-FCV-63-41 and -42.

4. "1C" cooling tower lift pump was reported to have no bearing lube water flow. We blew out the bearing lube water lines to establish good flow and the instrument section repaired a flowmeter.

5. Rebuilt the "2A-A" boric acid transfer pump, replacing the bearing, shaft, and seals.

6. Installed drain papers in the unit #2 lower ice condenser after the lower inlet doors were blown open and remained open for 2 to 3 hours.

7. Made repairs to the unit #1 main generator. Sent the rotor and bearing (#11) to Muscle Shoals to be remachined. Shipped the permanent magnet generator (PMG) to Westinghouse to replace the magnets and have it magnetized. After reinstalling the rotor, #11 bearing, and the coupling, bolt #17 in the coupling galled. Repulled the rotor and stator and replaced the galled bolt.

8. Replaced a broken Masoneilan Camflex operator on 1-FCV-3-329A with a spare operator borrowed from unit #2.

9. Installed Jimmy plates over door latches on eight doors in the control and auxiliary buildings per NRC requirements.

10. Repaired relief valve 1-VLV-67-582D which was broken at the nipple on the upper containment cooler "D".

11. Rebuilt the "2B-B" high pressure fire protection pump replacing the coupling and shaft which were found to be broken.

12. Furmanited Non-CSSC valves on systems #1, 3, and 6 (both units) and manways #19 and 23 on the MSR piping on unit #2. Furmanited CSSC valves 2-VLV-3-361A and 2-VLV-1-150.

13. Performed section XI maintenance on 1-FCV-1-182 by replacing the Bonnet gasket.

Plant Maintenance Summary

Electrical Maintenance

DATE.	COMPONENT.....	FAILURE DESCRIPTION.....	CAUSE OF FAILURE.....	CORRECTIVE ACTION ...	PRO#....
04-18	1-MTRB-082-1A/2	CHECK LUBE OIL CIRCULATING PUMP CONTROL TRANSFORMER PRIMARY LEADS FOR GOOD CLEARANCE	PREVENTATIVE MAINTENANCE	INSPECTED PRIMARY LEADS AND HAS GOOD CLEARANCE	NONE
05-03	1-FSV-001-0022	REMOVE GROUND ON VITAL BATTERY BOARD 1 A4B AND B4B	ROOF LEAKED AND WATER DRIPPED INTO JUNCTION BOX	CLEANED WATER OUT OF JUNCTION BOX 2042	NONE
05-05	1-ROD-085-SCPN 5	TEMPORARY CABLE WAS USED UNTIL ORIGINAL CABLE COULD BE REPAIRED AND REINSTALLED	N5 CABLE CONNECTOR WAS FULL OF BORATED WATER CAUSING CABLE TO GROUND AND HAVE RESISTANCE IN COIL	REPLACED CRDM N5 CABLE IN PLACE OF TEMPORARY CABLE	NONE
05-06	0-TCV-67-019/	VALVE WILL NOT OPEN ALL THE WAY	DIRTY	CLEANED VALVE	NONE
05-06	1-FSV-313-0223	REMOVE GROUND ON VITAL BATTERY BOARD 1 FUSE D3	WATER INTRUSION	CLEANED TERMINAL STRIP AND REPLACED DIODE CLEANED LIMIT SWITCH AND VALVE	NONE
05-08	1-XFD-31C-0904	BLOWN FUSE LINK	ACTIVATED PYROTRONICS PANEL	REPLACED LINKS	NONE
05-08	1-XFD-313-0905	REPLACE FUSABLE LINKS	ACTIVATED BY PYROTRONICS PANEL	REPLACED FUSABLE LINKS	NONE
05-09	2-MTRB-082-2A/3	CHECK LUBE OIL CIRCULATING PUMP CONTROL TRANSFORMER PRIMARY LEADS FOR GOOD CLEARANCE	PREVENTATIVE MAINTENANCE	INSPECTED PRIMARY LEADS . HAD GOOD CLEARANCE.	NONE
05-09	2-MTRB-082-2A/2	CHECK LUBE OIL CIRCULATING PUMP CONTROL TRANSFORMER PRIMARY LEADS FOR GOOD CLEARANCE	PREVENTATIVE MAINTENANCE	INSPECTED PRIMARY LEADS. HAS GOOD CLEARANCE	NONE
05-09	1-MTRB-082-1A/3	CHECK LUBE OIL CIRCULATING PUMP CONTROL TRANSFORMER PRIMARY LEADS FOR GOOD CLEARANCE	PREVENTATIVE MAINTENANCE	INSPECTED PRIMARY LEADS AND FOUND GOOD CLEARANCE	NONE

DATE.	COMPONENT.....	FAILURE DESCRIPTION.....	CAUSE OF FAILURE.....	CORRECTIVE ACTION.....	PRO#....
05-10	2-HTCK-234-223 P	CHECK MAIN DRAIN LINE TEMPERTURE CONTROLLER AND FUSES ON CIRCUIT 223 P.R.	FAULTY TEMPERTURE CONTROLLER	REPLACED TEMPERTURE CONTROLLER	NONE
05-10	0-BATB-250-QVJ	OXIDATION ON BATTERIES	PREVENTIVE MAINTANCE	CLEANED OXIDATION FROM BATTERIES	NONE
05-10	1-MTRB-082-1B/2	CHECK LUBE OIL CIRCULATING PUMP CONTROL TRANSFORMER PRIMARY LEADS FOR GOOD CLEARANCE	PREVENTATIVE MAINTENANCE	INSPECTED PRIMARY LEADS, HAD GOOD CLEARANCE	NONE
05-10	2-MTRB-082-028 3	CHECK LUBE OIL CIRCULATING PUMP CONTROL TRANSFORMER PRIMARY LEADS FOR GOOD CLEARANCE	PREVENTATIVE MAINTENANCE	INSPECTED CONTROL TRANSFORMER PRIMARY LEADS, HAD GOOD CLEARANCE	NONE
05-11	0-BATB-250-0QX T	CLEAN OXIDATION ON BATTERIES 12,19,3,9 AND 56	PREVENTATIVE MAINTENANCE	CLEANED BATTERIES	NONE
05-11	0-BATB-250-0QW T	CLEAN OXIDATION OFF BATTERIES 2,10 AND 42	PREVENTATIVE MAINTEANCE	CLEANED BATTERIES	NONE
05-11	0-BATB-250-0QY T	CLEAN OXIDATION OFF BATTERIES 3,4,8,16,31,41,45 AND 58	PREVENTATIVE MAINTENANCE	CLEANED BATTERIES	NONE
05-23	1-MVOP-001-1B-B	ISOLATION VALVE STEAM FLOW TO AUX. FWP NOT WORKING PROPERLY	TORQUE SWITCH OUT OF ADJUSTMENT	ADJUSTED TORQUE SWITCH PER MI 11.2 OPERATIONS STROKED VALVE DEMSTRATED OPERABILITY OF VALVE RETURNED TO SERVICE	1-84-187
05-23	1-FCV-062-0085	CHARGING FLOW RCS CONTROL POSITION SWITCH LIMIT LIGHTS WERE NOT SHOWING ACTUAL VALVE POSITION	ACTUATOR ARM WAS LOOSE ON SHAFT PREVENTING LIMITS FROM MAKING UP	ADJUSTED ACTUATOR ARM AND CHECKED FOR PROPER OPERATION	NONE
05-23	0-LS-032-90-B	AUX AIR COMPRESSOR B-B LOW OIL LEVEL SWITCH	DURING MAINTANCE OF OTHER EQUIPMENT THIS SWITCH WAS	REPLACED OIL LEVEL SWITCH	NONE

DATE.	COMPONENT.....	FAILURE DESCRIPTION.....	CAUSE OF FAILURE.....	CORRECTIVE ACTION.....	PRO#....
		BROKEN DURING MAINTANCE	BROKEN		
05-23	1-FCV-001-11-T	STEAM GENERATOR #2 MAIN STEAM HEADER ISOL. VLV. POSITION SWITCH INDICATOR LIGHT WOULD NOT COME ON WHILE IN #2 TEST POSITION	ACTUATOR ARM LOOSE ON SHAFT PREVENTING SWITCH FROM MAKING CONTACT	ADJUSTED ACTUATOR ARM	NONE
05-23	1-FCV-001-0004	STEAM GENERATOR #1 MAIN STEAM HEADER ISOLATION VALVE INDICATOR LIGHT WOULD NOT COME ON WHILE TEST POSITION WAS DEPRESSED	SET SCREW ON MICRO-SWITCH ARM WAS LOOSE PREVENTING SWITCH FROM MAKING CONTACT	ADJUSTED MICRO-SWITCH ARM	NONE
05-23	1-FCV-001-0007	STEAM GENERATOR #1 BLOWDOWN HEADER FLOW POSITION INDICATOR ARM WOULD NOT STRIKE LIMIT SWITCHES	ACTUATOR ARM STEM WAS BENT PREVENTING LIMIT SWITCHES FROM MAKING UP	STRAIGHTENED STEM ON ACTUATOR ARM	NONE
05-23	1-LCV-003-0148	STEAM GENERATOR #3 AUXILARY FEED WATER LEVEL CONTROL VALVE LIMIT LIGHTS WERE NOT SHOWING ACTUAL VALVE POSITION	POSSIBLY ACTUATOR ARM WAS LOOSE ON SHAFT PREVENTING LIMITS FROM MAKING UP	ADJUSTED LIMITS AND CHECKED FOR PROPER OPERATION	NONE
05-23	1-FCV-001-0025	STEAM GENERATOR #3 BLOWDOWN FLOW CONTROL VALVE LIMIT LIGHTS WERE NOT SHOWING ACTUAL VALVE POSITION	VALVE STEM WAS LOOSE CAUSING LIMIT SWITCH ACTUATOR ARM TO TURN AWAY FROM LIMIT SWITCH	ADJUSTED VALVE STEM AND CHECKED FOR PROPER OPERATION	1-84-185
05-23	1-INVA-250-ORQ	PREFERED INVERTER #1 EXHAUST FAN WILL NOT RUN	LOWER RIGHT BEARING BAD CAUSING MOTOR TO GO BAD	REPLACED MOTOR CHECKED ALIENMENT RETURNED TO SERVICE	NONE
05-23	1-MTRB-065-74	ANNULUS VACUUM CONTROL FAN 1-B HAD GROUND ON COMPONET 5D-1B1-B C&A VENT BOARD	WINDING IN MOTOR SHORTED OUT CAUSING DEAD GROUNDIN C-PHASE	REMOVED MOTOR AND HAD REWOUND AND REINSTALLED	NONE
05-23	1-INVB-250-OGL	120V AC VITAL INVERTER	BEARING WORN OUT CAUSING	REPLACED MOTOR AND WIND	NONE

DATE.	COMPONENT.....	FAILURE DESCRIPTION.....	CAUSE OF FAILURE.....	CORRECTIVE ACTION.....	PRO#....
		1-I FAN #1 MOTOR WAS STALLED WOULD NOT TURN	MOTOR TO BIND BURNING UP	SWITCH CHECKED ALIGNEMENT RETURNED TO SERVICE	
05-29	1-FCV-001-183A	STEAM GENERATOR #3 BLOWDOWN ISOL. VLV. INSIDE CONTAINMENT WOULD NOT SHOW PROPER VALVE POSITION BOTH RED AND GREEN LIGHTS WOULD SHOW WHEN VALVE WAS IN EITHER POSITION	BLOCK ON OPERATOR STEM THAT MAKES LIMITS TURNED ENOUGH THAT NEITHER LIMIT WAS BEING MADE PROBABLE CAUSE VIRBRATION	PUT BLOCK BACK IN PROPER POSITION CHECKED STEM ON OPERATOR FOR LOOSENESS RETURNED TO SERVICE	NONE
05-29	1-HS-003-148A	STEAM GENERATOR #3 VALVE SWITCH FAILED TO OPERATE	SNAP RING ON THUMB SWITCH HAD POPPED OFF HANDLE PROBABLE CAUSE OF NOT BEING PUT ON PROPERLY	REINSTALLED SNAP RING CHECKED FUNCTION OF SWITCH AND RETURNED TO SERVICE	NONE
05-31	1-FSV-061-0097	ON THE 125V BATTERY BOARD #II PANEL WAS SHOWING GROUND ONGLYCOL INLET ISOLATION VALVE 1-FCV-61-97	WIRES HOLDING LIMIT SWITCH CONTACTS OPEN ALSO GROUNDED CABLE	REARRANGED WIRES IN LIMIT SWITCH SPLICED CABLE AND CLEARED GROUND RETURNED TO SERVICE	NONE
05-31	1-INVB-250-ONE	BLOWN FUSES	SURGE OF POWER DUE TO UNIT TRIP	REPLACED FUSES FU2 FU2A FU13 FU14 CHECKED POWER SOURCE OK	1-84-200

32 records listed.

Instrument Maintenance

1. During monthly testing of UHI level switches, we found all switches to be within Technical Specification Tolerance and only two switches on unit 1 out of the desired tolerance of ±2" H_2O. We have now prepared a DCR to replace these switches due to the unreliability that has been experienced.

2. The No. 3 Heater Drain Tank Level Control Valves 1 and 2, LCV-6-106A & B, were set for split range operation instead of the parallel operation they had previously been set to. This improved the stability of the No. 3 Heater Drain Tank flow and therefore helped to improve the feedwater flow and steam generator level control.

UNIT 1

1. Completed the NIS power range calibrations required as a result of startup testing for the new core.

2. Completed all required initial calibrations for the reactor vessel level indication system. Returned to service on May 29, 1984.

3. During the month there were two (2) failures of the containment sump level transmitters as follows:

   a. 1-LT-63-179 failed on May 10, reading greater than 6 percent deviation during performance of SI-2. Mechanics found the output to be approximately 6 percent high over full range. Transmitter was recalibrated and returned to service. No "top off" was performed. (MR #A282500, PRO #1-84-188) (Curtis Lagasse)

   b. 1-LT-63-179 failed on May 21, reading low approximately 2 percent. Mechanics found the output of the transmitter to be approximately 0.7 mA low over entire range. Transmitter was recalibrated by exercising the "span" and "zero" adjustments and returned to service. Possible oxide buildup on the "zero" and "span" pots. No "top off" was performed. (MR #A283141, PRO #1-84-198) (R. G. Lewis).

4. During the month there were four (4) ABI's attributed to spurious signals. One (1) on the RM-90-101 monitor during a transfer of voltage on the same power supply and the other three (3) from the high background levels of the spent fuel pit. There was one combination ABI and CVI on unit 1 during the loss of a vital inverter when the reactor tripped.

   Additional modifications are being requested by CATEGORY D FCR 2406 to operate with a upper-level discrimator set below the max preamplifier pulse height.

Instrument Maintenance

UNIT 2

1. Rod position indicator H-6 was erratic and indicating a high resistance in the connector. Repair was achieved by applying voltage to the field cable and connector.

2. Replaced the MOOG Servo valve on the turbine throttle valve No. 3 and verified correct stroke.

3. Completed the modification to the square root converters on safety related flow loops to install a voltage regulation circuit.

4. The Terry Turbine tripped on turbine overspeed during a SI performance. A problem was found with SC-46-57 and FIC-46-57. Replaced the dropping resistor and returned the turbine to service.

INSTRUMENT MAINTENANCE MONTHLY SUMMARY 06-06-84 PAGE 1

MR.COMP	U	FUNC	SYS	ADDRESS.	COMP DATE....	DESCRIPTION.....	CORRECTIVE ACTION.....
A097553	2	FI	003	488	05/25/84	2-FI-003-488, INDICATING IMPROPERLY.	UPON INVESTIGATION THE FLOW TRANSMITTER WAS FOUND OUT OF CALIBRATION DUE TO NATURAL DRIFT. THE INDICATOR WAS RECALIBRATED AND RETURNED TO SERVICE.
A110942	1	FCV	003	48	05/25/84	1-FCV-003-48, VALVE DOESN'T RESPOND PROPERLY AT 30% LOAD AND WILL NO GO IN AUTO.	PACKING TOO TIGHT ON VALVE STEM. ADJUSTED PACKING CLEANED PILOT VALVE ON POSITIONER REPLACED AIR SUPPLY REGULATOR, & RESTORED VALVE.
A233327	1	FT	063	91A	06/01/84	1-FT-063-91A, 1-FT-63-91A, REPLACE TUBE FITTINGS, VENT PLUGS AND TUBING AS REQIRED TO ELIMINATE LEAKS	UPON INVESTIGATION THE TUBING FITTINGS WERE FOUND LOOSE DUE TO CYCLIC FATIGUE. THE FITTINGS WERE TIGHTENED AND TRANSMITTER WAS LEFT IN SERVICE.
A238091	1	PCV	001	5	05/03/84	1-PCV-001-5, WITH HS 1-6 IN AUTO AND 1-PIC-1-6A IN AUTO, THE POSITION INDICATING LIGHTS SHOWED RED AND GREEN (ON THEIR OWN). *NPRD*	THE GASKET ON THE POSITIONER WAS FOUND BAD DUE TO CYCLIC FATIGUE CAUSING THE AIR LEAK. REPLACED THE GASKET, ADJUSTED THE CURRENT TO PRESSURE CONVERTER AND RETURNED TO SERVICE.
A238242	0	O2R	043	5000	05/07/84	0-O2R-043-5000, O2 ANALYZER SHOWING < 0%, THERE SHOULD BE SOME KIND OF A READING. INVESTIGATE AND REPAIR.	INDICATOR LAMPS BAD & OUT OF ADJUSTMENT. READJUSTED & REPLACED LAMPS.
A245631	1	PCV	001	12	05/04/84	1-PCV-001-12, VALVE POSITIONER IS LEAKING AIR, BAD GASKET ON REG. WHERE AIR COMES IN. *NPRD*	THE GASKET ON THE POSITIONER WAS FOUND TO BE BAD DUE TO CYCLIC FATIGUE. REPLACED THE GASKET AND RETURNED TO SERVICE.
A247061	1	LCV	003	171	05/10/84	1-LCV-003-171, TO LOOP #4 S/G DOES NOT CONTROL AT SETPOINT, CONTINUES TO FEED S/G AFTER LEVEL IN S/G INCREASES PAST SETPOINT. INVESTIGATE AND REPAIR.	THE SET POINT DIAL WAS OUT OF CALIBRATION DUE TO DRIFT. RECALIBRATED THE SET POINT DIAL, VERIFIED PROPER OPERATION AND RETURNED TO SERVICE.
A247614	2	FT	003	48A	05/17/84	2-FT-003-48A, PERFORM CHANNEL CALIBRATION ON THE F-3-48A LOOP.	THE TRANSMITTER WAS FOUND OUT OF CALIBRATION DUE TO NATURAL DRIFT. RECALIBRATED THE TRANSMITTER, VERIFIED CALIBRATION OF THE CURRENT TO CURRENT MODULE VERIFIED OPERATION AND RETURNED TO SERVICE.
A247615	2	FT	003	90A	05/17/84	2-FT-003-90A, PERFORM CHANNEL CALIBRATION ON THE F-3-90 LOOP.	A PROBLEM WAS REPORTED WITH THIS LOOP. A COMPLETE LOOP CHECK WAS MADE AND NO PROBLEM WAS FOUND.
A247616	2		003	488	05/18/84	2--003-488, PERFORM CALIBRATION OF THE U2 F-3-488 FEEDWATER FLOW LOOP.	UPON INVESTIGATION THE FLOW TRANSMITTER WAS FOUND OUT OF CALIBRATION DUE TO NATURAL DRIFT. THE TRANSMITTER WAS RECALIBRATED AND RETURNED TO SERVICE.

INSTRUMENT MAINTENANCE MONTHLY SUMMARY 06-06-84 PAGE 2

MR.	COMP	U	FUNC	SYS	ADDRESS.	COMP DATE....	DESCRIPTION.....	CORRECTIVE ACTION.....
A247622	2		FI	003	48A	05/24/84	2-FI-003-48A, INDICATOR READING HIGHER THAN "B" CHANNEL VERIFY CAL.	INDICATOR OUT OF CALIBRATION. RECALIBRATED FI-3-48A.
A281530	2		FI	003	358	05/20/84	2-FI-003-358, INDICATOR READS LOW PER CHANNEL CHECK TRIP ASSOCIATED BISTABLES PER INSTRUCTION (TI-67), INVESTIGATE AND REPAIR AS REQUIRED AND RETURN TO NORMAL.	THE FLOW TRANSMITTER WAS FOUND OUT OF CALIBRATION CAUSING THE FAILURE. RECALIBRATED THE TRANSMITTER, BLED DOWN THE ASSOCIATED SENSE LINES AND RETURNED TO SERVICE.
A281543	2		PT	003	1	05/30/84	2-PT-003-1, PT FAILED HIGH CAUSING FEED PUMP TO BACK DOWN IN AUTO - INVESTIGATE & REPAIR	WATER HAD CONDENSED INTO JUNCTION BOX AT TRANSMITTER FROM A STEAM LEAK OVERHEAD. DRIED OUT JUNCTION BOX, REPLACED TEST BLOCK, & RECALIBRATED TRANSMITTER.
A281544	2		PI	001	33	05/30/84	2-PI-001-33, INDICATOR READING HIGH	TRANSMITTER OUT OF CAL. RECAL TRANSMITTER
A281646	1		FI	003	163	05/31/84	1-FI-003-163, FI SHOWS 40 GPM WITH A AFW PUMP OFF	FT-3-163 OUT OF CAL. RECAL. FT-3-163.
A281647	1		FI	003	155	05/31/84	1-FI-003-155, FLOW INDICATOR SHOWS 90 GPM WITH A AFW PUMP OFF	THE FLOW TRANSMITTER WAS FOUND OUT OF CALIBRATION DUE TO SETPOINT DRIFT. RECALIBRATED THE TRANSMITTER, BACK FILLED THE SENSE LINES AND RETURNED TO SERVICE.
A281752	1		TS	068	331	05/16/84	1-TS-068-331, RECALIBRATE TO ITS NORMAL SETPOINT OF 140 DEGREES F. THE SETPOINT WAS CHANGED PER TACF 1-84-062-68, MR A281755 WILL CLEAR TACF.	THE SWITCH WAS OUT OF CALIBRATION TO 210F WHICH WAS BRINGING IN THE ALARM IN THE CONTROL ROOM DUE TO CALIBRATION DRIFT. RECALIBRATED THE SWITCH TO 140F AND RETURNED TO SERVICE.
A281753	1		TS	068	330	05/16/84	1-TS-068-330, RECALIBRATE TO ITS NORMAL SETPOINT OF 140 DEGREES F. THE SETPOINT WAS CHANGED PER TACF 1-84-062-68. MR A281755 WILL CLEAR TACF.	THE SWITCH WAS OUT OF CALIBRATION TO 210F WHICH WAS BRINGING IN THE ALARM IN THE CONTROL ROOM DUE TO CALIBRATION DRIFT. RECALIBRATED THE SWITCH TO 140F AND RETURNED TO SERVICE.
A281754	1		TS	068	329	05/16/84	1-TS-068-329, RECALIBRATE TO IT NORMAL SETPOINT OF 140 DEGREES F. THE SETPOINT WAS CHANGED PER TACF 1-84-062-68. MR A281755 WILL CLEAR TACF.	THE SWITCH WAS OUT OF CALIBRATION TO 210F WHICH WAS BRINGING IN THE ALARM IN THE CONTROL ROOM DUE TO CALIBRATION DRIFT. RECALIBRATED THE SWITCH TO 140F AND RETURNED TO SERVICE.
A281755	1		TS	068	328	05/16/84	1-TS-068-328, RECALIBRATE TO ITS NORMAL SETPOINT OF 140 DEGREES F. CLEAR TACF 1-84-062-68 WHEN MR'S A281755, A281752, A281753, AND A281754 ARE COMPLETED.	THE SWITCH WAS OUT OF CALIBRATION TO 210F WHICH WAS BRINGING THE ALARM IN THE CONTROL ROOM DUE TO CALIBRATION DRIFT. RECALIBRATED THE SWITCH TO 140F AND RETURNED TO SERVICE.
A282045	1		FM	003	488	05/31/84	1-FM-003-488, LOOP 2 FW FLOW CONVERTER, LOW SIGNAL CUTOFF CIRCUIT IS STICKING. *NPRD*	IT WAS FOUND THAT THE COIL AND REED SWITCHES WERE BAD CAUSING THE FAILURE.

INSTRUMENT MAINTENANCE MONTHLY SUMMARY 06-06-84 PAGE 3

MP.COMP	U	FUNC	SYS	ADDRESS.	COMP DATE....	DESCRIPTION.....	CORRECTIVE ACTION.....
							THE REED SWITCHES AND COIL WERE REPLACED, PROPER OPERATION WAS VERIFIED , RECALIBRATED AND THE MODIFIER WAS RETURNED TO SERVICE.
A282092	2	TS	062	245	05/04/84	2-TS-062-245,CALIBRATE TEMP. SWITCH HIGH SETPOINT IS 175 DEGREES, INDICATION IS 177 DEGREES. *NPRD*	IT WAS FOUND THAT THE SETPOINTS ON THE SWITCH WERE OUT OF TOLERANCE DUE TO NATURAL DRIFT. THE TEMPERATURE SWITCH WAS RECALIBRATED AND RETURNED TO SERVICE.
A282323	1	LR	003	43P002	05/18/84	1-LR-003-43P002,GREEN PEN IS FAILED HI, REPAIR AS REQUIRED.	THE VALVE WHICH PERMITTED FLOW TO THE HIGH SIDE OF TRANSMITTER WAS FOUND CLOSED CAUSING THE FAILURE. OPENED THE VALVE, VERIFIED OPERATIONS AND RETURNED TO SERVICE.
A283101	1	TI	068	319	05/14/84	1-TI-068-319,RCS PREZ HAS HIGH ALARM ON TI-68-319 FOR LIQUID TEMPERATURE INDICATOR.	ON INVESTIGATION ON THE TEMPERATURE MODIFIER HAD DRIFTED HIGH DUE TO NATURAL DRIFT. RECALIBRATED THE MODIFIER AND RETURNED TO SERVICE.
A283141	1	LI	063	179	05/23/84	1-LI-063-179,LEVEL INDICATOR FAILED LOW, REPAIR.	THE TRANSMITTER WAS OUT OF CALIBRATION DUE TO NATURAL DRIFT WHICH CAUSED THE LEVEL INDICATOR TO FAIL LOW. THE LEVEL TRANSMITTER WAS RECALIBRATED AND RETURNED TO SERVICE.
A285343	1	LT	068	339	05/07/84	1-LT-068-339,VERIFY CALIBRATION OF 1-LT-68-339 PER IMI-99, CC 5.48. RECALIBRATE AND/OR REPAIR IF NECESSARY.	THE LEVEL TRANSMITTER WAS FOUND OUT OF CALIBRATION DUE TO NATURAL DRIFT. THE LEVEL TRANSMITTER WAS RECALIBRATED AND RETURNED TO SERVICE.
A286504	1	PCV	001	5	05/03/84	1-PCV-001-5,AIR IS BLOWING FROM GASKET AREA OF THE AIR REGULATOR FOR THIS PORV. *NPRD*	THE GASKET ON THE POSISTIONER WAS FOUND BAD DUE TO CYCLIC FATIGUE CAUSING THE OIL LEAK. REPLACED THE GASKET, ADJUSTED THE CURRENT TO PRESSURE CONVERTER AND RETURNED TO SERVICE.
A286641	0	RM	090	118	05/04/84	0-RM-090-118,RAD MONITOR APPEARS TO HAVE A SHORT. FAILS TO FUNCTION.	BAD POWER SUPPLY IN RP-30 MODULE. REPAIRED POWER SUPPLY.
A292031	1	FCV	003	48	05/25/84	1-FCV-003-48,CHECK S/G LEVELS FOR LOOPS 1, 3, & 4. *NPRD*	THE LOCAL REMOTE SWITCH WAS FOUND IN LOCAL POSITION AND SHOULD HAVE BEEN IN REMOTE CAUSING THE READINGS TO BE INCORRECT. PLACED SWITCH IN REMOTE POSITION AND RETURNED TO SERVICE
A292035	0	RM	090	225	05/25/84	0-RM-090-225,THE MODULE IS READING HIGHER THAN THE RECORDER.	CONTACTS DIRTY ON RECORDER INPUT. CLEANED CONTACTS ON RECORDER.

INSTRUMENT MAINTENANCE MONTHLY SUMMARY 06-06-84 PAGE 4

MR.COMP	U	FUNC SYS	ADDRESS.	COMP DATE....	DESCRIPTION.....	CORRECTIVE ACTION.....
A292048	1	FIS 001	55	05/29/84	1-FIS-001-55,FIS WAS FAILED HIGH	BELLOWS IN FIS PUNCTURED. REPLACED BELLOWS & RECALIBRATED.

31 records listed.

INSTRUMENT MAINTENANCE MONTHLY SUMMARY 06-06-84 PAGE 1

MR2....	U	FUNC	SYS	ADDRESS.	COMP DATE....	DESCRIPTION.....	CORRECTIVE ACTION.....
A242335	2	FT	003	48A	05/09/84	2-FT-003-48A, PERFORM CALIBRATION AND VERIFY OUTPUT WITH ROSEMOUNT TEST TRANSMITTER.	THE CAUSE WAS FROM NATURAL CALIBRATION DRIFT. THE TRANSMITTER WAS RECALIBRATED AND RETURNED TO SERVICE.
A283059	2	LCV	003	164	05/11/84	2-LCV-003-164, CANNOT GET FULL CLOSURE INDICATION ON LCV-3-164 WHEN SETPOINT IS DECREASED TO ZERO. REPORTED THAT AIR IS BLOWING @ VLV. REF. SI276	THE CONTROLLER WAS FOUND WITH A BAD MILLI VOLT TO CURRENT CIRCUIT BOARD CAUSING IT NOT TO GO TO ITS FULL OUTPUT. THE CIRCUIT BOARD WAS REPAIRED, VERIFIED CALIBRATION ON THE CONTROLLER AND RETURNED TO SERVICE.
A283060	2	LCV	003	164A	05/11/84	2-LCV-003-164A, CANNOT GET FULL CLOSURE INDICATION ON LCV-164A WHEN SETPOINT DECREASED TO ZERO. REPORTED THAT AIR IS BLOWING @ VLV. REF. SI276 INVESTIGATE & CORRECT TO SERVICE	THE MILLIVOLT TO CURRENT CIRCUIT BOARD WAS FOUND BAD CAUSING THE CONTROLLER NOT TO GIVE IT'S FULL OUTPUT VALUE. REPAIRED THE CIRCUIT BOARD, VERIFIED CALIBRATION AND RETURNED TO SERVICE.

3 records listed.

Plant Maintenance Summary

(Continued)

Field Services Group

1. ECN 2768--Reactor Pressure Vessel Level Indication System (RVLIS) (Unit 2)

   The mechanical workplan investigation and preparation began and the electrical preparation is continuing. The procurement of all electrical materials is underway.

2. ECN 2780--Post Accident Sampling Facility (Unit 2)

   All the mechanical materials have been received and the electrical equipment procurement process is continuing. The conduit work outside containment is underway while the workplan preparation is continuing. The following mechanical work is complete: HVAC system piping, hangers, tubing panels, and core drilling. The fire protection workplan is in the review cycle.

3. ECN 5198--Technical Support Center (TSC) (Unit 2)

   The modification of the status monitoring system (SMS) cabinets continued for the interface equipment. The control building conduit installation on elevation 685 continued. Both the SMS and elevation 685 continued. Both the SMS and elevation 685 conduit work should be completed by mid June. the conduit in the annulus and the auxiliary instrument room is complete. The cable pulling workplan preparation is near completion.

4. ECN--6055 Wide Range Pressure Transmitter to the RVLIS Panel

   The mechanical workplan investigation and preparation is in progress. The conduit installation is continuing in the auxiliary building.

5. ECN--5194 Iodine Monitoring Building

   The electrical workplan for unit 2 conduit is in the approval cycle. The termination of the unit 1 sample room air conditioning cable was completed. The mechanical workplan has been approved.

6. Nu Reg 0588 Items

   The unit 2 vent motor replacement has begun for non outage work (ECN-5370). No work has started on the solenoid valve replacement (ECN-5457). The workplan for the revision on the level switches was completed and the cable pull is in progress (ECN-5765). The workplan to replace the system 3 pressure switches was written and turned in for PORC review (ECN-5823). The investigation and preparation of the electrical workplan for valve operator replacement is in progress, units 1 & 2 (ECN-5824). The workplan writing is in progress for the limit switch replacement (ECN-5881), the flow switch replacement (ΣCN-5883) and is complete for the temperature switch replacement on system 12 and 30 (ECN-5882).

6. Nu Reg 0588 Items (Continued)

The flow transmitter replacement materials are on order from ENDES for a delivery of July 9, 1984 and the workplan preparation is in progress (ECN-5884). The relays for the revision of system 43 limit switches have arrived for unit 1 & 2 (ECN-5898). System 3 and 7 motors are on order for valve operators (ECN-5970). The miscellaneous instrument replacement workplan preparation is complete and is in the review cycle (ECN-5995). Also, the mechanical and electrical workplan preparations are in progress for the hydrogen monitor modification (ECN-6032).

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TENNESSEE VALLEY AUTHORITY

Sequoyah Nuclear Plant
P. O. Box 2000
Soddy-Daisy, Tennessee 37379

June 15, 1984

Nuclear Regulatory Commission
Office of Management Information
and Program Control
Washington, DC 20555

Gentlemen:

Enclosed is the May 1984 Monthly Operating Report to the NRC for Sequoyah Nuclear Plant.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

P. R. Wallace
Plant Manager

Enclosure
cc (Enclosure):

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