

NRC FORM 366 (12-81) 10 CFR 50		U.S. NUCLEAR REGULATORY COMMISSION LICENSEE EVENT REPORT		APPROVED BY OMB 3150-0011	
CONTROL BLOCK:		(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)			
01	M A P P S I	02	10000000000000	03	411111
LICENSEE CODE		LICENSE NUMBER		LICENSE TYPE CAT 58	
CON'T	01	R E P O R T S U R C E L	D O C K E Y N U M B E R 050 - 0293	E V E N T D A T E 021479	R E P O R T D A T E 061384
EVENT DESCRIPTION AND PROBABLE CONSEQUENCES					
02	On February 14, 1979, while performing Surveillance Test 8.5.4.1 (HPCI Pump Operability and Flow Rate Test @ 1000 psig), the HPCI turbine tripped.				
03	Immediate investigation revealed that the lower head gasket on the gland seal condenser had ruptured. The HPCI was declared INOP, and operability of backup systems was immediately demonstrated. The event created no threat to the public health and safety.				
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05					
06					
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08					
09	S F	X	Z	X X X X X X X	Z Z
SYSTEM CODE		CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	VALVE SUBCODE
17	LER/RP REPORT NO. 79	SEQUENTIAL REPORT NO. 008	OCCURRENCE CODE /	REPORT TYPE X	REVISION NO. 1
ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS	ATTACHMENT SUBMITTED
A	Z	Z	Z	00000	Y
NPRD-S PRIME COMP. SUPPLIER	COMPOUND MANUFACTURER				
N	Z	Z	Z	Z	Z
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS					
10	Probable cause was attributed to excessive moisture entering an electrical box,				
11	resulting from a ruptured gland seal condenser gasket, causing the inadvertent trip of the aux. oil pump. Repairs were completed, test 8.5.4.1 successfully performed, and HPCI was returned to service. A similar occurrence was identified in LER 82-24.				
12					
13					
14					
15	E	% POWER 100	OTHER STATUS N/A	B	DISCOVERY DESCRIPTION Operator Observation
FACILITY STATUS		AMOUNT OF ACTIVITY	CORRECTIVE ACTIONS	LOCATION OF RELEASE	
Z	Z	N/A		N/A	
PERSONNEL EXPOSURES					
17	000	TYPED DESCRIPTION	N/A		
NUMBER					
PERSONNEL INJURIES					
18	000	DESCRIPTION	N/A		
NUMBER					
LOSS OF OR DAMAGE TO FACILITY					
19		TYPE DESCRIPTION	N/A		
TYPE					
PUBLICITY ISSUED					
20	N	DESCRIPTION	N/A		
ISSUED					

NAME OF PREPARER P.J. Hamilton PHONE (617) 746-7900

ATTACHMENT TO LER 79-008/03X-1

On February 14, 1979, while performing Surveillance Test 8.5.4.1 (HPCI Pump Operability Flow Rate and Valve Test @ 1000 psig), the HPCI turbine tripped. Immediate investigation revealed that the lower head gasket on the gland seal condenser had ruptured. The HPCI was declared inoperable, and operability of the backup system was immediately demonstrated.

The most probable cause of the turbine trip is attributed to the inadvertent trip of the auxiliary oil pump prior to the turbine reaching speed. It was determined at the time, that the gland seal condenser head gasket had ruptured due to excessive pressure, resulting from the out-of-calibration condition of Pressure Control Valve PCV 2301-46. Excessive moisture, caused by the ruptured gasket, entered an electrical control circuit box causing the inadvertent trip of the auxiliary oil pump.

Immediate corrective action involved installation of new gland seal condenser head gaskets, and recalibration of the pressure control valve. Surveillance Procedure 8.5.4.1 was satisfactorily performed, and the HPCI was declared operable on February 14, 1979.

To reduce the potential for future gasket failures due to pressure transients, Restricting Orifice RO-2301-60 was relocated to the inlet side of the gland seal condenser from the outlet side (refer to GE SIL No. 129). Also, to reduce the frequency of gasket failures, the upper and lower neoprene gaskets were removed and replaced with gaskets made of Chesterton 290 material.

A similar occurrence was identified in LER 82-24.

RECEIVED
FEB 24 1979
U.S. NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20545

BOSTON EDISON COMPANY
800 BOYLSTON STREET
BOSTON, MASSACHUSETTS 02199

WILLIAM D. HARRINGTON
SENIOR VICE PRESIDENT
NUCLEAR

June 13, 1984

BECO Ltr. #84-081

Dr. Thomas E. Murley
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

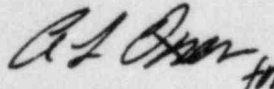
Docket Number 50-293
License DPR-35

Dear Sir:

The attached update Licensee Event Report 79-008/03X-1, HPCI Trip During Testing, is hereby submitted in accordance with the requirements of Pilgrim Nuclear Power Station Technical Specification 6.9.B.2.b.

If there are any questions on this subject, please do not hesitate to contact me.

Respectfully submitted,



W. D. Harrington

PH:ko

Enclosure: LER 79-008/03X-1

cc: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

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