

OPERATING DATA REPORT

DOCKET NO. 50-289
 DATE June 15, 1984
 COMPLETED BY C. Smyth
 TELEPHONE (717) 948-8551

OPERATING STATUS

1. UNIT NAME: THREE MILE ISLAND UNIT 1
2. REPORTING PERIOD: MAY, 1984.
3. LICENSED THERMAL POWER (MWT): 2535.
4. NAMEPLATE RATING (GROSS MWE): 871.
5. DESIGN ELECTRICAL RATING (NET MWE): 819.
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 824.
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 776.

NOTES

8. IF CHANGES OCCUR IN (ITEMS 3-7) SINCE LAST REPORT, GIVE REASONS:

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE)

10. REASONS FOR RESTRICTIONS, IF ANY:

	THIS MONTH	YR-TO-DATE	CUMMULATIVE
11. HOURS IN REPORTING PERIOD	744.	3647.	85440.
12. NUMBER OF HOURS REACTOR WAS CRITICAL	0.0	0.0	31731.8
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	838.5
14. HOURS GENERATOR ON-LINE	0.0	0.0	31180.9
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED (MWH)	0.	0.	76531071.
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	0.	0.	25484330.
18. NET ELECTRICAL ENERGY GENERATED (MWH)	0.	0.	23840053.
19. UNIT SERVICE FACTOR	0.0	0.0	36.5
20. UNIT AVAILABILITY FACTOR	0.0	0.0	36.5
21. UNIT CAPACITY FACTOR (USING MDC NET)	0.0	0.0	35.6
22. UNIT CAPACITY FACTOR (USING DER NET)	0.0	0.0	34.1
23. UNIT FORCED OUTAGE RATE	100.0	100.0	60.1

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

8407020005 840615
 PDR ADOCK 05000289
 R PDR

IE 24
 1/1

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
UNIT TMI-1
DATE June 15, 1984
COMPLETED BY C. Smyth
TELEPHONE (717) 948-8551

MONTH: MAY

DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)
1	0.
2	0.
3	0.
4	0.
5	0.
6	0.
7	0.
8	0.
9	0.
10	0.
11	0.
12	0.
13	0.
14	0.
15	0.
16	0.

DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)
17	0.
18	0.
19	0.
20	0.
21	0.
22	0.
23	0.
24	0.
25	0.
26	0.
27	0.
28	0.
29	0.
30	0.
31	0.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH MAY

DOCKET NO. 50-289
 UNIT NAME TMI-1
 DATE June 15, 1984
 COMPLETED BY C. W. Smyth
 TELEPHONE (717) 048-8551

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
1	84/01/01	F	744	D	1	N/A	ZZ	ZZZZZZ	Regulatory Restraint Order

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

⁵
 Exhibit I - Same Source

OPERATIONS SUMMARY

The unit was shutdown the entire month by order of the NRC. Core cooling was provided by the Decay Heat Removal System except as described below. The following evolutions were performed during the month to support Hot Functional Testing:

- 5/7/84 - Started Circ Water System.
- 5/9/84 - Filled Condensate System.
- 5/14/84 - Partially drained both OTSGs.
- 5/16/84 - Established seal injection to RCPs.
 - Established RCS cleanup.
 - Commenced pressurizing RCS to 312 psig.
 - Started up Condensate System.
 - Established vacuum in condensers.
- 5/21/84 - Set containment integrity.
- 5/22/84 - Commenced RCS heatup.
- 5/23/84 - RCS at 525°F, 2155 psig.

The following testing was performed as part of Hot Functional Testing:

- . 8th stage heaters operational hydro.
- . RCS leak rate measurement (new procedure and new computer program).
- . Hanger expansion tests for head vents and high point vents.
- . HPI cross-connect flow check.
- . PORV retest (refurbished valve and modified spring assembly).
- . Reactor coolant pump vibration analyses and setting.
- . Operational hydro for:
 - .. RCP Pump 1B,
 - .. All OTSG primary manways,
 - .. Decay heat drop line tap,
 - .. Head vent,
 - .. DHV-1,
 - .. Main steam relief valves setting.

No major problems were identified during this testing. On May 26, 1984 RCS cooldown was commenced and the plant remained in cold shutdown on Decay Heat Removal for the remainder of the month.

A Reactor Building Integrated Leak Rate Test was performed on 5/8/84. The test met the acceptance criteria with very few problems encountered.

MAJOR SAFETY RELATED MAINTENANCE

During the month the following major maintenance items were completed.

Reactor Coolant Pump Balancing

RC-P1A/B/C & D were satisfactorily balanced following repair work on RC-P1B, and realignment of RC-P-1A, C & D.

Local Leak Rate Testing

Leak Rate Valves LR-V2 and 3, the Reactor Building Fire Service and the OTSG Drain Penetrations were tested satisfactorily. Intermediate Cooling Valve IC-V3 failed its leak test and was tested satisfactorily following repairs to the valve seat and gate.

Component Retorquing

During HFT, bolts were retorqued on the OTSG manway and handhole covers as were bonnets on pressure seal valves CF-V1A/B, CF-V4A/B, DH-V1, 2 and 22A.

Main Steam Safety Valve Testing

All main steam safety valves were tested satisfactorily during HFT.

Waste Gas Pump Overhaul

WG-P1A was overhauled following failure to pump to rated capacity. The failure was attributed to moisture related corrosion and erosion of internals. Following repairs the pump was tested with satisfactory results.

ESAS Relay Replacement

All LSAS relays were replaced because of the unreliability of the previously installed Clark relays. This work was performed to resolve problems identified in LER 83-24.

Atmospheric Dump Valves Replaced

MS-V4A/B were replaced with a different model (Fisher Valves) because of a history of leaking by. Prior to installation the internals were modified. Work also included valve operator inspection, weld NDT, flush and hydro testing.

REFUELING INFORMATION REQUEST

1. Name of Facility:

Three Mile Island Nuclear Station, Unit 1

2. Scheduled date for next refueling shutdown:

Unknown

3. Scheduled date for restart following refueling:

Unknown

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

If answer is yes, in general, what will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?

Amendment No. 50, Cycle 5 reload, was approved on 3-16-79.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

N/A

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

N/A

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool:

(a) 177

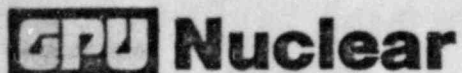
(b) 208

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 752. There are no planned increases at this time.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

1988 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).



GPU Nuclear Corporation
Post Office Box 480
Route 441 South
Middletown, Pennsylvania 17057-0191
717 944-7621
TELEX 84-2386
Writer's Direct Dial Number:

June 15, 1984
5211-84-2145

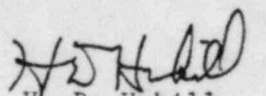
Office of Management Information and P. Control
Attn: W. C. McDonald
c/o Distribution Services Branch DPC, ADM
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Mr. McDonald:

Three Mile Island Nuclear Station, Unit I (TMI-1)
Operating License No. DPR-50
Docket No. 50-289
May Monthly Operating Report

Enclosed please find two (2) copies of the May Operating Report for Three Mile Island Nuclear Station Unit 1.

Sincerely,


H. D. Hukill,
Director, TMI-1

HDH:mle
Attachments

cc: V. Stello
Dr. E. T. Murley

DESIGNATED ORIGINAL
Certified By MR Beebe 06/29/84

IE24
11