

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Point Beach Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 3 0 1 1										PAGE (3) 1 OF 0 2		
TITLE (4) Inadvertent Actuation of Emergency Safeguards																						
EVENT DATE (5)			LER NUMBER (6)					REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)											
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES						DOCKET NUMBER(S)							
									None						0 5 0 0 0 0 0 0 0 0 0 0 0 0 0 0							
0 5	1 9	8 4	8 4	0 0 3	0 0 0	6	2 1	8 4	None						0 5 0 0 0 0 0 0 0 0 0 0 0 0 0 0							
OPERATING MODE (9) N		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11)																				
POWER LEVEL (10) 0 0 1 0		20.402(b)					20.405(c)					<input checked="" type="checkbox"/> 50.73(a)(2)(iv)					73.71(b)					
		20.405(a)(1)(i)					50.36(c)(1)					50.73(a)(2)(v)					73.71(c)					
		20.405(a)(1)(ii)					50.36(c)(2)					50.73(a)(2)(vii)					OTHER (Specify in Abstract below and in Text, NRC Form 366A)					
		20.405(a)(1)(iii)					50.73(a)(2)(ii)					50.73(a)(2)(viii)(A)										
		20.405(a)(1)(iv)					50.73(a)(2)(ii)					50.73(a)(2)(viii)(B)										
		20.405(a)(1)(v)					50.73(a)(2)(iii)					50.73(a)(2)(ix)										
LICENSEE CONTACT FOR THIS LER (12)																						
NAME C. W. Fay, Vice President-Nuclear Power										TELEPHONE NUMBER AREA CODE 4 1 4 2 7 7 2 8 1 1												
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																						
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS												
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR						
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO										

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

While performing a reactor shutdown and partial cooldown, an inadvertent safety injection (SI) actuation occurred while decreasing reactor coolant system pressure. The operator failed to block SI prior to the actuation setpoint. The reactor was shut down prior to the actuation.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1) Point Beach Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 3 0 1 8 4 - 0 0 3 - 0 0 0 2 OF 0 2	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			

TEXT: (If more space is required, use additional NRC Form 366A's) (17)

While performing a scheduled reactor shutdown and partial cooldown for maintenance, an inadvertent safety injection (SI) actuation occurred while decreasing reactor coolant system pressure. The operator performing the depressurization recognized that the SI signals must be blocked prior to 1750 psi but less than 1765 psi. At that point, he asked another operator to block SI, and as the operator was proceeding to do it, SI actuated.

SI was allowed to sequence per design and after 2 minutes, SI signals were reset. All actuated systems were returned to normal lineup from the actuation a short time later. No water was injected into the primary system because the reactor coolant system pressure remained greater than the shutoff head for the SI pumps. All systems and equipment operated as designed. The reactor was shut down prior to the SI actuation.

A discussion with the operator indicated that he realized his mistake immediately. A review of the applicable procedures was also conducted and it was concluded that the procedures are adequate. This event is being discussed with all operators in order to prevent future occurrences. No further action is considered to be required.



Wisconsin Electric POWER COMPANY
231 W. MICHIGAN, P.O. BOX 2046, MILWAUKEE, WI 53201

DMB

June 21, 1984

Mr. J. G. Keppler, Regional Administrator
Office of Inspection and Enforcement,
Region III
U. S. NUCLEAR REGULATORY COMMISSION
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Dear Mr. Keppler:

DOCKET NO. 50-301
LICENSEE EVENT REPORT NO. 84-003-00
INADVERTENT ACTUATION OF EMERGENCY SAFEGUARDS
POINT BEACH NUCLEAR PLANT, UNIT 2

Enclosed is Licensee Event Report No. 84-003-00 which provides a description of an inadvertent actuation of emergency safeguards while in a shutdown condition reportable in accordance with 10 CFR 50.73(a)(2)(iv), "Any event or condition that resulted in manual or automatic actuation of any engineered safety feature, including the reactor protection system."

Very truly yours,

Vice President-Nuclear Power

C. W. Fay

Enclosure

Copy to NRC Resident Inspector

JUN 25 1984

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