

ENCLOSURE

PLANT HATCH - UNIT 1
NRC DOCKET 50-321
OPERATING LICENSE DPR-57
REQUEST TO REVISE TECHNICAL SPECIFICATIONS;
PROTECTIVE INSTRUMENTATION

PAGE CHANGE INSTRUCTIONS

The proposed change to the Plant Hatch Unit 1 Technical Specifications will be incorporated as follows:

Remove Page

3.2-4

Insert Page

3.2-4

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Notes for Table 3.2-1

- a. The column entitled "Ref. No." is only for convenience so that a one-to-one relationship can be established between lines in Table 3.2-1 and items in Table 4.2-1.
- b. Primary containment integrity shall be maintained at all times prior to withdrawing control rods for the purpose of going critical, when the reactor is critical, or when the reactor water temperature is above 212°F and fuel is in the reactor vessel except while performing low-power physics tests at atmospheric pressure at power levels not to exceed 5 MWt, or performing an inservice vessel hydrostatic or leakage test.

When primary containment integrity is required, there shall be two operable or tripped trip systems for each function.

When performing inservice hydrostatic or leakage testing on the reactor vessel with the reactor coolant temperature above 212°F, reactor vessel water level instrumentation associated with the low low (Level 2) trip requires two operable or tripped channels. The drywell pressure trip is not required because primary containment integrity is not required.

- c. If the number of operable channels cannot be met for one of the trip systems, the inoperable channel(s) or the associated trip system shall be tripped. However, one trip signal channel of a trip system may be inoperable for up to 2 hours during periods of required surveillance testing without tripping the channel or associated trip system, provided that the other remaining channel(s) monitoring that same parameter within that trip system is (are) operable.
- d. The valves associated with each Group isolation are given in Table 3.7-1.
- e. Within 24 hours prior to the planned start of the hydrogen injection test with the reactor power at greater than 20% rated power, the normal full-power radiation background level and associated trip setpoints may be changed based on a calculated value of the radiation level expected during the test. The background radiation level and associated trip setpoints may be adjusted during the test based on either calculations or measurements of actual radiation levels resulting from hydrogen injection. The background radiation level shall be determined and associated trip setpoints shall be set within 24 hours of re-establishing normal radiation levels after completion of hydrogen injection and prior to establishing reactor power levels below 20% rated power.
- f. The high differential flow signal to the RWCU isolation valves may be bypassed for up to 2 hours during periods of system restoration, maintenance, or testing.