

## LICENSEE EVENT REPORT

EXHIBIT A

NRC Form 366  
(9-83)

U.S. Nuclear Regulatory Commission  
Approved OMB No. 3150-0104  
Expires: 8/31/85

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)										DOCKET NUMBER (2) (PAGE 3)											
Arkansas Nuclear One - Unit 2										10151010101316181101012											
TITLE (4)																					
Primary Overcurrent Protection Device on Containment Penetration Inoperable - LCO Exceeded																					
EVENT DATE (5)					LER NUMBER (6)					REPORT DATE (7)					OTHER FACILITIES INVOLVED (8)						
Month		Day		Year		Sequential Number		Revision Number			Month		Day		Year		Facility Names				
01	91	01	5	7	8	81	4	--	0	1	2	--	0	0	01	6	1	8	81	4	
															Docket Number(s)						
															10151010101						
															10151010101						
OPERATING MODE (9) 1 THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8:																					
(Check one or more of the following) (11)																					
POWER LEVEL		20.402(b)					20.405(c)					50.73(a)(2)(iv)					73.71(b)				
(10) 11010		20.405(a)(1)(i)					50.36(c)(1)					50.73(a)(2)(v)					73.71(c)				
		20.405(a)(1)(ii)					50.36(c)(2)					50.73(a)(2)(vii)					Other (Specify in				
		20.405(a)(1)(iii)					X 50.73(a)(2)(i)					50.73(a)(2)(viii)(A)					Abstract below and				
		20.405(a)(1)(iv)					50.73(a)(2)(ii)					50.73(a)(2)(viii)(B)					in Text, NRC Form				
		20.405(a)(1)(v)					50.73(a)(2)(iii)					50.73(a)(2)(x)					366A)				
LICENSEE CONTACT FOR THIS LER (12)																					
Name															Telephone Number						
Dan Moeggenberg, Licensing Engineer															Area						
															Code						
															510119161431213161						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																					
Cause/System		Component		Manufacturer		Reportable to NPRDS		Cause/System		Component		Manufacturer		Reportable to NPRDS							
SUPPLEMENT REPORT EXPECTED (14)																					
YES (If yes, complete Expected Submission Date) [X] NO															EXPECTED SUBMISSION DATE (15)						
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)																					

During a review of documentation to verify the correctness of a proposed technical specification change request regarding containment penetration overcurrent protection devices, an apparent error was noted in a vendor electrical drawing for reactor coolant system sampling cabinet 2C116. The drawing indicated that fuse protection, being relied upon as the primary penetration overcurrent protection device, was bypassed by jumpers. Since no other documentation was found that indicated the jumpers had been removed subsequent to receipt and installation of the cabinet, a special investigation was performed on 5/18/84, at 1620 hours to ascertain the existence of the jumpers. The jumpers were found to exist as indicated on the drawings. An engineering evaluation was promptly performed to allow removal of the jumpers, and the necessary wiring changes were completed by 1800 hours on 5/18/84.

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PDR ADCK 05000368  
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(9-83)

Form 1062.01B  
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (e)	PAGE (3)				
		<table border="1"> <tr> <th data-bbox="1015 436 1131 478">Sequential</th> <th data-bbox="1131 436 1247 478">Revision</th> </tr> <tr> <td data-bbox="1015 478 1131 521">Year</td> <td data-bbox="1131 478 1247 521">Number</td> </tr> </table>	Sequential	Revision	Year	Number	
Sequential	Revision						
Year	Number						
Arkansas Nuclear One - Unit 2	0151010101 31 61 81	81 41 -- 0 1 2 -- 0 1 0	01210F1012				

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 5/18/84, a special inspection was performed to resolve an apparent drawing error on a vendor print. The print indicated hard wired jumpers around power and control fusing to three DC powered solenoid valves in the reactor coolant (RCS) sampling system. The jumpers were contained in 2C116, RCS sampling cabinet, supplied by Delphi Industries. After visually verifying that the jumpers did in fact exist as shown by the print, an engineering evaluation was promptly conducted and rewiring instructions were provided to the maintenance staff. The jumpers were removed, restoring fuse protection, within approximately two hours. This is considered an isolated event, therefore, no additional action to reduce the probability of future occurrences is planned.

The fusing is relied upon as the primary device to provide overcurrent protection for containment penetration 2WR26-3. With the fusing in the circuit bypassed by the jumpers, 2D21 DC motor control center breaker 26 was still available to interrupt possible fault current to prevent damage to containment penetration 2WR26-3.

Since there is no record of a plant change to install the jumpers, it is believed that the jumpers were installed either during the fabrication or installation of the cabinet. As such, the error is considered a design/construction error that has existed since 1978. The event date, September 5, 1978, is the date when the unit first entered Mode 4 and Technical Specification 3.8.2.5 first became applicable. Because the Limiting Condition for Operation associated with Technical Specification 3.8.2.5 was exceeded, this event is reportable per 50.73(a)(2)(i)(b).

Another event regarding containment overcurrent protective devices (breakers) was reported in (50-368) LER 83-049/01T-0.



ARKANSAS POWER & LIGHT COMPANY

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June 18, 1984

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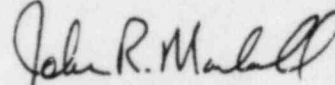
U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Subject: Arkansas Nuclear One - Unit 2  
Docket No. 50-368  
License No. NPF-6  
Licensee Event Report  
No. 84-012-00

Gentlemen:

In accordance with 10CFR50.73(a)(2)(i), attached is the subject report concerning an apparent error noted in a vendor electrical drawing for reactor coolant system sampling cabinet 2C116.

Very truly yours,

  
John R. Marshall  
Manager, Licensing

JRM:JRS:ac

Attachment

cc: Mr. Norman M. Haller, Director  
Office of Management & Program Analysis  
U. S. Nuclear Regulatory Commission  
Washington, DC 20555

Mr. Richard C. DeYoung  
Office of Inspection and Enforcement  
U. S. Nuclear Regulatory Commission  
Washington, DC 20555

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