

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
Susquehanna Steam Electric Station

DOCKET NUMBER (2)

0 5 0 0 0 3 8 8

PAGE (3)

1 OF 0 2

TITLE (4)

LCO Action Not Met.

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)				
0	5	2	5	8	4	8	4	0	0	7	0	5	0	0	0
0	5	2	5	8	4	8	4	0	0	6	2	2	8	4	0

OPERATING MODE (9)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following): (11)
2	20.402(b)
POWER LEVEL (10)	20.406(a)(1)(i)
0	20.406(a)(1)(ii)
0	20.406(a)(1)(iii)
2	20.406(a)(1)(iv)
	20.406(a)(1)(v)
	20.406(c)(1)
	20.406(c)(2)
	20.406(c)(2)(i)
	20.406(c)(2)(ii)
	20.406(c)(2)(iii)
	50.73(a)(2)(iv)
	50.73(a)(2)(v)
	50.73(a)(2)(vii)
	50.73(a)(2)(viii)(A)
	50.73(a)(2)(viii)(B)
	50.73(a)(2)(ix)
	73.71(b)
	73.71(c)
	OTHER (Specify in Abstract below and in Text, NRC Form 308A)

LICENSEE CONTACT FOR THIS LER (12)
NAME
L.A. Kuczynski - Nuclear Plant Specialist III
TELEPHONE NUMBER
AREA CODE
7 1 1 7 5 1 4 2 1 - 1 3 7 1 5 1 9

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	
A	CIE	*	*	N						

SUPPLEMENTAL REPORT EXPECTED (14)
YES (If yes, complete EXPECTED SUBMISSION DATE) ☐ NO ☒
EXPECTED SUBMISSION DATE (15)
MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On May 25, 1984, a Reactor Water Cleanup differential pressure switch was jumpered out for a calibration check. Due to a personnel oversight, when the associated Technical Specification time limit expired, the required ACTION (close the affected system isolation valves) was not completed. The ACTION was taken approximately one hour late. Shift Supervision has been counseled on this occurrence.

8406280337 840622
PDR ADOCK 05000388
S PDR

*Not Applicable

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1) Susquehanna Steam Electric Station Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 3 8 8 8 4 - 0 0 7 - 0 0 0 2 OF 0 2	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			

TEXT (If more space is required, use additional NRC Form 366A's) (17)

The trip channels for the Reactor Water Cleanup (RWCU) Containment Inboard and Outboard Isolation valves each contain a high differential pressure (dp) switch to close its respective valve in the event of a RWCU pipe break. At 0000 on May 25, 1984, the 'A' high dp switch was jumpered out for a calibration check. (The 'A' switch provides its signal to the inboard isolation valve.) This resulted in less than the minimum required operable channels for the 'RWCU Flow-High' trip function and LCO 3.3.2 ACTION b became applicable. The associated footnote permits two hours to elapse before invoking the Action of Table 3.3.2-1. The high dp switch was not returned to service within two hours, per ACTION 23, the RWCU isolation valves were to be closed within one hour and the system declared inoperable. Contrary to the above, ACTION 23 was not completed until 0400, one hour past the maximum allowed. Shift Supervision (licensed, utility personnel), anticipating the imminent return of the high dp switch, inadvertently exceeded the Action requirements. Shift Supervision has been counseled on this occurrence. Emphasis has been placed on the need to foresee possible delays and take action when required, not to rely on commitments by work groups.



Pennsylvania Power & Light Company

Two North Ninth Street • Allentown, PA 18101 • 215 / 770-5151

June 22, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 84-007-00
ER 100450 FILE 841-23
PLA-2240

Docket No. 50-388
License No. NPF-22

Attached is Licensee Event Report 84-007-00. This event was determined reportable per 10CFR50.73(a)(2)(i) in that Reactor Water Cleanup valves were not closed within the time limit specified by Technical Specification 3.3.2.

H.W. Keiser
Superintendent of Plant-Susquehanna

LAK/pjg

cc: Dr. Thomas E. Murley
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
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