

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)  
Susquehanna Steam Electric Station - Unit 1DOCKET NUMBER (2)  
0 5 0 0 0 3 8 7 1 OF 0 2TITLE (4)  
Core Spray Valve Isolation Signal

EVENT DATE (6)			LER NUMBER (8)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)											
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)									
0	5	1	6	8	4	3	4	0	2	6	0	0	0	6	1	5	8	4	SSES - Unit 2	0 5 0 0 0 3 8 8
												0 5 0 0 0								

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)									
POWER LEVEL (10)	1 0 0	20.402(b)		20.405(e)		50.73(a)(2)(iv)		73.71(b)			
		20.405(a)(1)(i)		50.38(a)(1)		50.73(a)(2)(v)		73.71(e)			
		20.405(a)(1)(ii)		50.38(a)(2)		50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 355A)			
		20.405(a)(1)(iii)	X	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)					
		20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)					
		20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(ix)					

LICENSEE CONTACT FOR THIS LER (12)  
NAME  
R.W. StanleyTELEPHONE NUMBER  
AREA CODE  
7 1 1 7 5 4 2 - 1 3 1 6 6

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
A	B	M	*	*	N				

SUPPLEMENTAL REPORT EXPECTED (14)  
YES (If yes, complete EXPECTED SUBMISSION DATE) ☐ NO ☒  
EXPECTED SUBMISSION DATE (15)  
MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

A Technical Specification review revealed the current plant design to the Core Spray Valve Full Flow Test Isolation Signal is not in agreement with the isolation signal specified by the Technical Specifications or the FSAR. The As Built condition is presently being modified to agree with the Technical Specification and FSAR.

8406280326 840615  
PDR ADOCK 05000387  
S PDR

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)  Susquehanna Steam Electric Station Unit 1	DOCKET NUMBER (2)  0 5 0 0 0 3 8 7 8 4 - 0 2 6 - 0 0 0 2 OF 0 2		LER NUMBER (8)			PAGE (3)	
			YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		

TEXT (If more space is required, use additional NRC Form 386A's) (17)

Technical Specification Table 3.6.3-1 and FSAR Table 6.2-12 require Core Spray Full Flow Test Isolation Valves HV-152F015A, HV-152F015B (Unit 1) and HV-252F015A, HV-252F015B (Unit 2) to isolate on

- 1) Reactor Vessel Low Level 1, or
- 2) Primary Containment High Drywell Pressure

The As Built condition isolates the valves properly on .

- 1) Reactor Vessel Low Level 1.

The design and the As Built condition do not agree with the Technical Specifications or the FSAR in that these documents require an isolation on

- 2) A Primary Containment High Drywell Pressure.

The design and As Built condition both isolate the valves on Primary Containment High Drywell Pressure with a Low Reactor Pressure Permissive Signal.

Administrative control was placed on the full flow test valves by yellow tagging the valves closed. This will prevent the opening of these valves without the shift supervisor's knowledge.

The Core Spray Full Flow Test Valve Isolation Signal non-conformance was noted during a Technical Specification review, by utility personnel. Non-conformance Report (NCR) 84-747 and 84-748 were written to document the isolation signal discrepancy. Plant Modification Request (PMR) 84-3085 and 84-3086 are issued to modify the isolation logic. The modification involves removing the Low Reactor Pressure Permissive from the circuit, therefore, allowing the Core Spray Full Flow Test Valve to isolate on either High Drywell Pressure or Low Reactor Vessel Water Level only.

This occurrence was noted on May 16, 1984 at 1715. Reactor power level on Unit 1 was 100%. Unit 2 was <1%

The discrepancy in the isolation signals for the Core Spray Full Flow Test Isolation Signal will not effect the safe operation of the Nuclear Power Plant since the Core Spray System operability is not effected.



Pennsylvania Power & Light Company

June 14, 1984

Two North Ninth Street • Allentown, PA 18101 • 215 / 770-5151

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION  
LICENSEE EVENT REPORT 84-026-00  
ER 100450 FILE 841-23  
PLA-2237

Docket No. 50-387  
License No. NPF-14

Attached is Licensee Event Report 84-026. This event was determined reportable per 10CFR50.73(a)(2)(i) in that an isolation signal to the Core Spray Full Flow Test was not per Technical Specifications.

H.W. Keiser  
Superintendent of Plant-Susquehanna

RWS/pjg

cc: Dr. Thomas E. Murley  
Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, PA 19406

Mr. R.H. Jacobs  
Senior Resident Inspector  
U.S. Nuclear Regulatory Commission  
P.O. Box 52  
Shickshinny, PA 18655

IE22  
1/1