NRC Form 366
(9/83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 3	0 5 0 0 0 3 6 2	1 OF 0 2

TITLE (4)

REACTOR TRIP BREAKER SERIAL NO. 29 UNDERVOLTAGE DEVICE ANOMALY

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQ. NUMBER	REV. NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
										0 5 0 0 0
0 5	2 5	8 4	8 4	0 1 8	0 0					0 5 0 0 0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
1		20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
1 0 0		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	X OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	Informational Report
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
J. G. HAYNES, STATION MANAGER	7 1 4 4 9 2 - 7 7 0 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

Abstract (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

This report is submitted to provide information concerning operation of Reactor Trip Breakers (RTB's) on their undervoltage (UV) trip devices. (As in the past, the breakers continue to function acceptably using the shunt trip device.) Although this occurrence was determined to be not reportable under the Unit 3 Technical Specifications or 10 CFR 50.73, we are submitting this report to inform you of the circumstances involved and corrective actions taken.

On May 25, 1984, with Unit 3 in Mode 1 at 100% power, surveillance testing in accordance with S023-II-11.161, "Reactor Breaker Undervoltage Response Time Testing," prior to the scheduled RTB maintenance, was in progress. During this surveillance, the UV trip device for RTB Serial No. 256A4002-656-29 exhibited a procedurally unacceptable response time. The breaker was replaced with a spare. The breaker will be refurbished (see LER 2-84-025, Docket No. 50-361).

Public health and safety were not affected since the breaker was located in the nonsafety-related cross-tie position (RTB #9) which does not open on a reactor trip. Furthermore, the breaker continues to function properly using the shunt trip device.

IE02

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQ. NUMBER	REV. NUMBER	
SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 3	0 5 0 0 0 3 6 2	8 4	- 0 1 8	- 0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

This report is submitted to provide information concerning operation of Reactor Trip Breakers (RTB's) (EIIS Component Code 52) on their undervoltage (UV) trip devices. (As in the past, the breakers continue to function acceptably using the shunt trip device.) Although this occurrence was determined to be not reportable under the Unit 3 Technical Specifications or 10 CFR 50.73, we are submitting this report to inform you of the circumstances involved and corrective actions taken.

On May 25, 1984, with Unit 3 in Mode 1 at 100% power, surveillance testing in accordance with S023-II-11.161, "Reactor Breaker Undervoltage Response Time Testing," prior to the scheduled RTB maintenance, was in progress. During this surveillance, the UV trip device for RTB #9 (Serial No. 256A4002-656-29) exhibited a procedurally unacceptable response time. The response times (in order) were 146 msec, 40 msec and 36 msec. S023-II-11.161 contains an acceptance criterion of 82 msec, which was developed from baseline testing and consideration of the Combustion Engineering (CE) guideline of 100 msec. The shunt trip feature operated properly. No Action Statements were entered since the RTB was located in the nonsafety-related cross-tie position and does not open on a reactor trip.

The breaker was replaced with spare breaker Serial No. 256A4002-656-45. Based on previous timing anomalies as described in Licensee Event Report (LER) 84-016, Docket No. 50-362, RTB Serial No. 256A4002-656-29 will be returned to the vendor (General Electric) for refurbishment as described in LER 84-025, Docket No. 50-361.

Our enhanced RTB surveillance and maintenance program trending has been successful in identifying potential problem RTB's. We are currently developing a program to include all RTB's for Units 2 and 3 into a refurbishment program which will include replacement of the trip shafts and latch roller bearings.

Public health and safety were not affected since the breaker was located in the nonsafety-related cross-tie position (RTB #9) which does not open on a reactor trip. Furthermore, the breaker continues to function properly using the shunt trip device.

Southern California Edison Company

SAN ONOFRE NUCLEAR GENERATING STATION
P.O. BOX 128
SAN CLEMENTE, CALIFORNIA 92672

J. G. HAYNES
STATION MANAGER

TELEPHONE
(714) 492-7700

June 21, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Docket No. 50-362
Informational Report
Licensee Event Report No. 84-018
San Onofre Nuclear Generating Station, Unit 3

This submittal provides an informational Licensee Event Report (LER) for an occurrence involving the Plant Protection System (PPS). The health and safety of plant personnel or the public were not affected by these occurrences.

If you require any additional information, please so advise.

Sincerely,

J. G. Haynes

Enclosure: LER 84-018

cc: A. E. Chaffee (USNRC Resident Inspector, Units 1, 2 and 3)
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

J. B. Martin (Regional Administrator, USNRC Region V)

Institute of Nuclear Power Operations (INPO)

IE22
1/1