

**LICENSEE EVENT REPORT (LER)**

FACILITY NAME (1) Washington Nuclear Plant - Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 3 9 7 1 OF 0 2										PAGE (3) 1																			
TITLE (4) Unscheduled Initiation of Control Room Emergency Filtration Units																																							
EVENT DATE (5)						LER NUMBER (6)						REPORT DATE (7)						OTHER FACILITIES INVOLVED (8)																					
MONTH		DAY		YEAR		YEAR		SEQUENTIAL NUMBER		REVISION NUMBER		MONTH		DAY		YEAR		FACILITY NAMES						DOCKET NUMBER(S)															
0 5		2 8		8 4		4 8		4 0		5 2		0 0		0 6		2 2		8 4								0 5 0 0 0													
OPERATING MODE (9) 3						THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following): (11)																																	
POWER LEVEL (10) 0 0 0						20.402(b)						20.406(a)						X 50.73(a)(2)(iv)						73.71(b)															
						20.406(a)(1)(i)						50.35(a)(1)						50.73(a)(2)(iv)						73.71(c)															
						20.406(a)(1)(ii)						50.36(a)(2)						50.73(a)(2)(vii)						X OTHER (Specify in Abstract below and in Text, NRC Form 366A)															
						20.406(a)(1)(iii)						50.73(a)(2)(i)						50.73(a)(2)(viii)(A)						50.72(b)(2)(ii)															
						20.406(a)(1)(iv)						50.73(a)(2)(ii)						50.73(a)(2)(viii)(B)																					
20.406(a)(1)(v)						50.73(a)(2)(iii)						50.73(a)(2)(ix)																											
LICENSEE CONTACT FOR THIS LER (12)																																							
NAME C.M. Powers, Reactor Engineering Supervisor																				TELEPHONE NUMBER 5 0 9 3 7 7 1 - 2 5 0 1																			
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																				Ext. 2996																			
CAUSE		SYSTEM		COMPONENT		MANUFACTURER		REPORTABLE TO NPDOS		CAUSE		SYSTEM		COMPONENT		MANUFACTURER		REPORTABLE TO NPDOS																					
B		I L		R A K		0 2 0		N																															
SUPPLEMENTAL REPORT EXPECTED (14)																				EXPECTED SUBMISSION DATE (15)										MONTH DAY YEAR									
YES (If "N" complete EXPECTED SUBMISSION DATE)																				X NO																			
ABSTRACT (Limit to 1400 spaces; use approximately 1/3 space single-space typewritten lines) (16)																																							
<p>A Control Room Emergency Filtration Unit (an ESF system) was automatically actuated due to a spike on the corresponding Control Room Outside Air Radiation Monitors.</p> <p>After verifying that radiation levels were not above normal background, the radiation monitors and emergency filtration units were reset and returned to normal operation.</p> <p>Subsequent investigation determined the cause of the spike to have been the closure of a reactor core isolation cooling valve (EPN: RCIC-V-13).</p>																																							
8406280142 840622 PDR ADOCK 05000397 S PDR																																							
IE22 1/1																																							

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Washington Nuclear Plant - Unit 2	0 5 0 0 0 3 9 7 8 4 -	0 5	2	--	0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 388A's) (17)

Plant Conditions

- a) Plant Power - 0%  
b) Plant Mode - 3

Event

The Control Room Emergency Filtration Unit (WMA-FN-54A) was automatically actuated on 5-28-84 by a High-High radiation alarm originating from the outside air intake monitors (EPN: WOA-RIS-31A & 32A).

Immediate Corrective Action

Normal background radiation levels were observed at the monitors. The associated radiation recorder (WOA-RR-31) revealed that monitors 31A and 32A had received a spurious spike of sufficient magnitude to trip the High-High alarm. These alarms were promptly reset and the ESF system returned to normal.

Notification was given to the NRC in accordance with the requirements of 10CFR50.72(b)(2)(ii).

Further Corrective Action

It was determined that the spike originated from the actuation of a reactor core isolation cooling valve (RCIC-V-13). A Plant modification was initiated to install noise suppression devices in the control circuitry of this valve. Investigation and resolution of noise problems are continuing on the Radiation Monitoring and interfacing systems.

Safety Significance

There were no safety consequences associated with this event and all Plant systems performed as required during the event.

## Washington Public Power Supply System

P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

Docket No. 50-397  
June 22, 1984

Document Control Desk  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

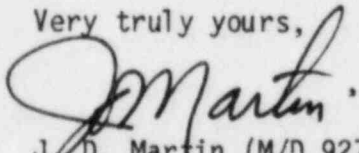
Subject: NUCLEAR PLANT NO. 2  
LICENSEE EVENT REPORT NO. 84-052

Dear Sir:

Transmitted herewith is Licensee Event Report No. 84-052 for WNP-2 Plant. This report is submitted in response to the report requirements of Technical Specification Section 6.9.1.7 and discusses the item of reportability, corrective action taken, and action taken to preclude recurrence.

This is the follow-up report to the verbal notification given at 1253 hours on May 28, 1984.

Very truly yours,



J. D. Martin (M/D 927M)  
WNP-2 Plant Manager

JDM:mm

Enclosure:  
Licensee Event Report No. 84-052

cc: Mr. John B. Martin, Administrator  
Region V, Office of Inspection and Enforcement  
U.S. Nuclear Regulatory Commission  
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Walnut Creek, California 94596  
Mr. A. D. Toth, NRC Resident Inspector (901A)  
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