

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) EDWIN I. HATCH, UNIT 2										DOCKET NUMBER (2) 0 5 0 0 0 3 6 6				PAGE (3) 1 OF 0 3										
TITLE (4) Safety Relief Valve on SLC																								
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)														
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)											
0	5	2	5	8	4	8	4	—	0	0	5	—	0	0	0	6	2	3	8	4	0 5 0 0 0			
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following) (11)																					
POWER LEVEL (10) 0 0 0			20.402(b)				20.406(c)				50.73(a)(2)(iv)				73.71(b)									
			20.406(a)(1)(i)				50.36(c)(1)				<input checked="" type="checkbox"/> 50.73(a)(2)(v)				73.71(c)									
			20.406(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)				OTHER: Specify in Abstract below and in Text, NRC Form 365A/									
			20.406(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)													
			20.406(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)													
			20.406(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(x)													
LICENSEE CONTACT FOR THIS LER (12)																								
NAME T. L. Elton, Acting Superintendent of Regulatory Compliance										TELEPHONE NUMBER 9 1 2 3 6 7 + 1 7 8 5 1														
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																								
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC														
X	B R	I R V L 2 6 5		Y																				
X	B R	I R V L 2 6 5		Y																				
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)				MONTH		DAY		YEAR				
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO												

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 5/25/84 during performance of the "STANDBY LIQUID CONTROL SYSTEM" procedure (HNP-2-6310), (with the reactor in cold shutdown) plant personnel determined that the Standby Liquid Control system's pressure relief valves lifted at a lower than expected pressure when they were bench tested. Consequently, in the event of a loss of coolant accident the standby liquid control system would not have injected its sodium pentaborate solution against a reactor vessel pressure in excess of 600 PSIG.

A design change was initiated which requires replacing the existing pressure relief valves with new pressure relief valves of a different model. One of the two pressure relief valves has been satisfactorily replaced. The second relief valve will be installed prior to loading fuel in the vessel.

8406280039 840623
PDR ADOCK 05000366
S PDRIE 22
111

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
EDWIN I. HATCH, UNIT 2	0 5 0 0 0 3 6 6	8 4	— 0 0 5	— 0 0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

This 30 day LER is required by 10CFR 50.73 (a)(2)(v)(A) because it showed that the Standby Liquid Control system might not have been capable of shutting down the reactor, and maintaining it in safe shutdown.

On 05/25/84, during performance of the six month "STANDBY LIQUID CONTROL SYSTEM" procedure, HNP-2-6310, (with the reactor in cold shutdown), plant personnel determined that the Standby Liquid Control (SLC) system's pressure relief valves for loop "A" and loop "B" lifted at a pressure lower than the expected 1350 PSIG \pm 25 PSIG when they were bench tested. Relief valve 2C41-F029A (loop A) lifted at 600 PSIG, and relief valve 2C41-F029B (loop B) lifted at 700 PSIG. These relief valves are positioned on the discharge side of the SLC pumps. When either valve lifts, the respective pump's discharge volume is then returned to the intake side of the SLC pump, thus completing a closed loop which recirculates the sodium pentaborate solution instead of injecting it into the reactor vessel.

In the event that a sufficient number of control rods could not have been inserted to control reactivity the SLC system would not have been able to inject into the reactor vessel against a pressure greater than 700 PSIG.

The control rod drive system was operable during this non-repetitive event. No justification for continued operation is required because the unit was in cold shutdown for recirculation piping repairs when this event occurred. This event had no impact upon any other system in Unit 2, or Unit 1.

After disassembly and examination it was determined that the valve stem in relief valve 2C41-F029A was broken at the cotter pin hole which allowed the valve spring to decompress and lose its tension. Relief valve 2C41-F029B was disassembled and found to have excessive wear on the valve stem and the valve stem guide.

A design change was initiated to replace both existing Lonergan model LCT 20 relief valves with Lonergan model LCT 30 relief valves. Relief valve 2C41-F029A (model LCT 30) was satisfactorily bench tested per HNP-2-6310 and installed on 06/08/84.

Relief valve 2C41-F029B (model LCT 30) has not been installed because it has not been received from the vendor. However, it will be tested per HNP-2-6310 and installed prior to loading fuel in the vessel.

When relief valve 2C41-F029B is received from the vendor, it will be bench tested per HNP-2-6310 and installed (promised date of vendor shipment is 06-20-84).

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

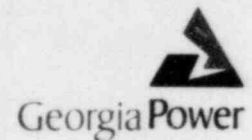
FACILITY NAME (1) EDWIN I. HATCH, UNIT 2	DOCKET NUMBER (2) 0 5 0 0 0 3 6 6 8 4 - 0 0 4 - 0 0 0 3 OF 0 3	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			

TEXT (If more space is required, use additional NRC Form 365A's) (17)

IDENTIFICATION OF EACH FAILED COMPONENT

MPL No.	Vendor	Model No.
2C41-F029A	LONERGAN	LCT-20
2C41-F029B	LONERGAN	LCT-20

Georgia Power Company
Post Office Box 439
Baxley, Georgia 31513
Telephone 912 367-7781
912 537-9444



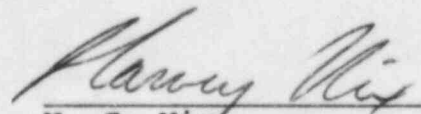
Edwin I. Hatch Nuclear Plant

June 23, 1984
GM-84-537

PLANT E. I. HATCH
Licensee Event Report
Docket No. 50-366

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Attached is Licensee Event Report No. 50-366/1984-05. This report is required by 10CFR 50.73(a)(2)(v)(A).


H. C. Nix
General Manager

see
HCN/TLE/vlt

xc: R. J. Kelly
R. E. Conway
J. T. Beckham, Jr.
P. D. Rice
K. M. Gillespie
Superintendent of Regulatory Compliance
R. D. Baker
Control Room
Document Control

IE22
1/1