



Commonwealth Edison  
LaSalle County Nuclear Station  
2601 N. 21st. Rd.  
Marseilles, Illinois 61341  
Telephone 815/357-6761

February 10, 1992

Director of Nuclear Reactor Regulation  
United States Nuclear Regulatory Commission  
Mail Station P1-137  
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering  
LaSalle County Nuclear Power Station for January, 1992.

Very truly yours,

  
G. J. Diederich  
for Station Manager  
LaSalle County Station

GJD/MJC/djf

Enclosure

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LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

JANUARY 1992

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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## I. INTRODUCTION (Unit 1)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit One was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761, extension 2427.

## II. MONTHLY REPORT

### A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

Day	Time	Event
1	0000	Reactor critical, Generator on-line at 930 Mwe.
	0500	Increased power level to 1090 Mwe.
	2300	Reduced power level to 850 Mwe due to system load.
2	0520	Increased power level to 1090 Mwe.
	2200	Reduced power level to 850 Mwe due to system load.
3	0500	Increased power level to 1100 Mwe.
7	0200	Reduced power level to 1070 Mwe for CRD exercising.
8	0000	Increased power level to 1130 Mwe.
12	2300	Reduced power level to 900 Mwe due to system load.
13	0330	Increased power level to 1080 Mwe.
	2300	Reduced power level to 920 Mwe for CRD exercising.
14	0500	Increased power level to 1080 Mwe.
	2200	Reduced power level to 750 Mwe to perform weekly and monthly surveillances.
15	0630	Increased power level to 1000 Mwe.
16	0530	Increased power level to 1130 Mwe.
	2330	Reduced power level to 850 Mwe due to system load.
17	0430	Increased power level to 1070 Mwe.
	1430	Increased power level to 1130 Mwe.
21	0000	Reduced power level to 950 Mwe for performance of Main Turbine testing.
	0410	Increased power level to 1100 Mwe.
	0900	Increased power level to 1130 Mwe.
24	0100	Reduced power level to 1000 Mwe due to system load.
	0430	Increased power level to 1100 Mwe.
	0730	Increased power level to 1130 Mwe.

## II. MONTHLY REPORT (CONTINUED)

### A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

<u>Day</u>	<u>Time</u>	<u>Event</u>
26	2300	Reduced power level to 1060 Mwe due to system load.
27	1000	Increased power level to 1130 Mwe.
	2200	Reduced power level to 850 Mwe due to system load and cycling of the Turbine Control Valves.
28	0500	Increased power level to 1100 Mwe.
	1100	Increased power level to 1130 Mwe.
	2330	Reduced power level to 1000 Mwe due to problems with the Heater Drain Pump forward valve (power level held due to system load).
29	0730	Increased power level to 1130 Mwe.
30	2230	Reduced power level to 850 Mwe to perform maintenance on the Heater Drain Pump forward valve.
31	0630	Increased power level to 1100 Mwe.
	2400	Reactor critical, Generator on-line at 1130 Mwe.



B. AMENDMENTS TO THE FACILITY LICENSE OR TECHNICAL SPECIFICATION

(None.)

C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY-RELATED EQUIPMENT (including SOR differential pressure switch failure reports).  
(See Table 1)

D. LICENSEE EVENT REPORTS (Unit 1)

<u>LER Number</u>	<u>Date</u>	<u>Description</u>
92-001-00	01/07/92	Average Power Range Monitors gain adjustment factors set nonconservatively due to a communication error during performance of Average Power Range Monitor Gain Adjustment surveillance.

E. DATA TABULATIONS (Unit 1)

1. Operating Data Report (See Table 2)
2. Average Daily Unit Power Level (See Table 3)
3. Unit Shutdowns and Significant Power Reductions (See Table 4)

C. TABLE 1 (Unit 1)

MAJOR CORRECTIVE MAINTENANCE TO  
SAFETY-RELATED EQUIPMENT

WORK REQUEST NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L12795	Reactor Core Isolation Valve packing leak. Cooling Turbine Suction Isolation Valve 1E51-F045		None.	Adjusted valve packing.
L13102	Main Steam Line Differential Temperature Switch 1E31-N615A	Temperature switch meter module.	Degraded operation.	Replaced meter module.

(No SOR Failures this month.)



TABLE 2  
E.1 OPERATING DATA REPORT

DOCKET NO. 859-376  
UNIT LASALLE ONE  
DATE February 10, 1992  
COMPLETED BY H.J. CIALKOWSKI  
TELEPHONE (815)-357-6761

OPERATING STATUS

1. REPORTING PERIOD: January 1992 GROSS HOURS IN REPORTING PERIOD: 744  
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3,323 MAX DEMAND CAPACITY (MWe-Net): 1,036  
DESIGN ELECTRICAL RATING (MWe-Net): 1,078  
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A  
4. REASONS FOR RESTRICTION (IF ANY):

REPORTING PERIOD DATA

	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	744.0	744.0	48,100.5
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,641.2
7. GENERATOR ON-LINE TIME (HOURS)	744.0	744.0	47,122.4
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1.0
9. THERMAL ENERGY GENERATED (MMBtu)	2,409,175	2,409,175	137,566,154
10. ELECTRICAL ENERGY GENERATED (MWe-Gross)	825,074	825,074	45,914,846
11. ELECTRICAL ENERGY GENERATED (MWe-Net)	800,830	800,830	43,995,975
12. REACTOR SERVICE FACTOR (%)	100.0	100.0	67.8
13. REACTOR AVAILABILITY FACTOR (%)	100.0	100.0	70.2
14. UNIT SERVICE FACTOR (%)	100.0	100.0	66.5
15. UNIT AVAILABILITY FACTOR (%)	100.0	100.0	66.5
16. UNIT CAPACITY FACTOR (USING RDC) (%)	103.9	103.9	59.9
17. UNIT CAPACITY FACTOR (USING DESIGN MWe) (%)	99.9	99.9	57.4
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	0.0	7.5

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

TABLE 3  
E.2 AVERAGE DAILY UNIT POWER LEVEL (MW<sub>e</sub>-Net)

DOCKET NO. 950-873  
UNIT LASALLE ONE  
DATE February 10, 1992  
COMPLETED BY M.J. CIALKOWSKI  
TELEPHONE (815)-357-6761

REPORT PERIOD: January 1992

DAY	POWER	DAY	POWER
1	1,029	17	1,037
2	1,006	18	1,103
3	1,023	19	1,106
4	1,090	20	1,102
5	1,090	21	1,078
6	1,099	22	1,110
7	1,088	23	1,100
8	1,099	24	1,088
9	1,090	25	1,110
10	1,099	26	1,100
11	1,102	27	1,079
12	1,097	28	1,029
13	1,055	29	1,087
14	1,052	30	1,099
15	957	31	1,041
16	1,090		

TABLE 4

E.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%  
(Unit 1)

YEARLY SEQUENTIAL NUMBER	DATE (YYMMDD)	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER/DVR # if applicable)
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(None.)

## SUMMARY OF OPERATION:

The Unit remained on-line at high power throughout the month. Several minor power reductions were required due to system loading, maintenance activities and routine surveillances.

F. UNIQUE REPORTING REQUIREMENTS (Unit 1)

1. Safety/Relief valve operations

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO &amp; TYPE ACTUATION</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
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(None)

2. ECCS System Outages  
(See Table 5)
3. Changes to the Off-Site Dose Calculation Manual  
(None.)
4. Major changes to Radioactive Waste Treatment Systems.  
(None.)
5. Indications of Failed Fuel Elements.  
(None.)

(Unit 1)  
Table 5

F.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE</u>
(U-0)		
0013	ODG01K	Replace cooling water pump.
0026	ODG08CA	Starting air compressor inspection.
0027	ODG08CB	Starting air compressor inspection.
(U-1)		
0022	1E51-C003	Replace water leg pump.
0033	1E51-C001	Lubrication.
0038	1E51-F019	Motor inspection.
0047	1E22-C302B	Perform inspection.
0052	1DG08CA	Starting air compressor inspection.
0053		



LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

JANUARY 1992

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18



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- F. UNIQUE REPORTING REQUIREMENTS
  - 1. Safety/Relief Valve Operations
  - 2. ECCS System Outages
  - 3. Off-Site Dose Calculation Manual Changes
  - 4. Major Changes to Radioactive Waste Treatment System
  - 5. Indications of Failed Fuel Elements

## I. INTRODUCTION (Unit 2)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit Two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on June 19, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761 extension 2427.

## II. MONTHLY REPORT

### A. SUMMARY OF OPERATING EXPERIENCE (Unit 2)

<u>Day</u>	<u>Time</u>	<u>Event</u>
1	0000	Reactor critical, Generator on-line at 995 Mwe Unit in coastdown.
2	1900	Power level at 980 Mwe, continuing coastdown.
3	1330	Power level at 850 Mwe, control rods being inserted per rod sequence.
	1420	Power level at 683 Mwe.
	1700	Power level at 535 Mwe.
	1800	Power level at 295 Mwe.
	2100	Power level at 116 Mwe.
4	0000	Power level at 63 Mwe, control rod insertions held.
	0105	Reduced power level to 10 Mwe via load set.
	0155	Generator offline, refueling outage (L2R04) began.
	1200	Reactor subcritical.
31	2400	Reactor subcritical, Generator off line, L2R04 in progress.

B. AMENDMENTS TO THE FACILITY LICENSE OR TECHNICAL SPECIFICATION

(None.)

C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY RELATED EQUIPMENT (including SOR differential pressure switch failure reports).  
(See Table 1)

D. LICENSEE EVENT REPORTS (Unit 2)

<u>LER Number</u>	<u>Date</u>	<u>Description</u>
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92-001-00	01/17/92	Inadvertant occurrence of a Primary Containment Group II isolation during performance of Reactor Vessel Low Water Level 1 & 2 Isolation Channels B & D Refuel Calibration surveillance.
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E. DATA TABULATIONS (Unit 2)

1. Operating Data Report.  
(See Table 2)
2. Average Daily Unit Power Level.  
(See Table 3)
3. Unit Shutdowns and Significant Power Reductions.  
(See Table 4)



TABLE 3  
E.1 OPERATING DATA REPORT

DOCKET NO. 950-374  
UNIT LASALLE TWO  
DATE February 10, 1992  
COMPLETED BY H. J. DIALKOWSKI  
TELEPHONE (815)-357-6761

OPERATING STATUS

1. REPORTING PERIOD: January 1992 GROSS HOURS IN REPORTING PERIOD: 744  
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3,323 MAX DEMAND CAPACITY (MWe-Net): 1,036  
DESIGN ELECTRICAL RATING (MWe-Net): 1,078

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASONS FOR RESTRICTION (IF ANY):

REPORTING PERIOD DATA

	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	84.0	84.0	45,019.8
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,716.9
7. GENERATOR ON-LINE TIME (HOURS)	73.9	73.9	44,276.5
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	0.0
9. THERMAL ENERGY GENERATED (MMWh)	180,986	180,986	132,423,385
10. ELECTRICAL ENERGY GENERATED (MMWh-Gross)	64,274	64,274	43,954,034
11. ELECTRICAL ENERGY GENERATED (MWe-Net)	54,161	54,161	42,185,665
12. REACTOR SERVICE FACTOR (%)	11.3	11.3	70.5
13. REACTOR AVAILABILITY FACTOR (%)	11.3	11.3	73.2
14. UNIT SERVICE FACTOR (%)	9.9	9.9	69.3
15. UNIT AVAILABILITY FACTOR (%)	9.9	9.9	69.3
16. UNIT CAPACITY FACTOR (USING MDC) (%)	7.0	7.0	63.7
17. UNIT CAPACITY FACTOR (USING DESIGN MWe) (%)	6.8	6.8	61.3
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	0.0	13.1

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):  
REFUELING (L2R04) 01/04/92 11 WEEKS

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

TABLE 3  
5.2 AVERAGE DAILY UNIT POWER LEVEL (HWe-Net)

DOCKET NO. 950-374  
UNIT LASALLE TWO  
DATE February 10, 1992  
COMPLETED BY M.J. CIALKOWSKI  
TELEPHONE (815)-357-6761

REPORT PERIOD: January 1992

DAY	POWER	DAY	POWER
1	961	17	-12
2	953	18	-12
3	653	19	-12
4	-5	20	-11
5	-12	21	-11
6	-12	22	-11
7	-11	23	-11
8	-11	24	-11
9	-11	25	-11
10	-11	26	-11
11	-11	27	-11
12	-11	28	-11
13	-11	29	-11
14	-12	30	-11
15	-12	31	-11
16	-12		



C. TABLE 1 (Unit 2)

MAJOR CORRECTIVE MAINTENANCE TO  
SAFETY-RELATED EQUIPMENT

<u>WORK REQUEST NUMBER</u>	<u>COMPONENT</u>	<u>CAUSE OF MALFUNCTION</u>	<u>RESULTS AND EFFECTS ON SAFE PLANT OPERATION</u>	<u>CORRECTIVE ACTION</u>
L12684	Division I 125 Volt Battery Volt Meter 2EI-DC057	Voltmeter found defective during surveillance.	Inaccurate voltage readings.	Replaced voltmeter.
L12952	Unit 2 '0' Diesel Generator Output Breaker ACB 2413	Test switch control.	None.	Replaced test switch control.

(No SOR Failures this month.)

TABLE 4

E.3 UNIT SHUTDOWNS AND POWER REDUCTIONS >20%  
(UNIT 2)

YEARLY SEQUENTIAL NUMBER	DATE (YYMMDD)	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER/LVR # if applicable)
01	920104	S	670.1	C	1	Refueling outage L2R04.

## SUMMARY OF OPERATION:

The unit entered the month in coastdown in preparation for the upcoming refuel outage. The generator was taken off line 01/04/92 and the unit entered a scheduled refuel outage.

F. UNIQUE REPORTING REQUIREMENTS (Unit 2)

1. Safety/Relief Valve Operations

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO &amp; TYPE ACTUATIONS</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
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(None.)

2. ECCS System Outages  
(See Table 5.)

3. Changes to the Off-Site Dose Calculation Manual.  
(None.)

4. Major changes to Radioactive Waste Treatment Systems.  
(None.)

5. Indications of Failed Fuel Elements.  
(None.)

(Unit 2)  
Table 5

F.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE</u>
223	2E21-C001	Perform inspection.
244	2E22-F012	Perform votes testing.
245	2E22-F023	Perform inspection.
246	2E22-F015	Perform votes testing.
249	2E22-F004	Perform votes testing.
553 554	2E22-S001	Replace battery charger.
856	2E12-F042A	Repair valve motor.