



J. Phillip Bayne
Executive Vice President
Nuclear Generation

June 15, 1984
JPN-84-37

Director of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Attention: Mr. Domenic B. Vassallo, Chief
Operating Reactors Branch No.2
Division of Licensing

Subject: James A. Fitzpatrick Nuclear Power Plant
Docket No. 50-333
NUREG-0737 Item II.E.4.1
Dedicated Hydrogen Penetrations

References: 1. PASNY letter, R.J. Pasternak to
T.A. Ippolito, dated September 30, 1980
(JAFP-80-759).
2. NYPA letter, J.P. Bayne to D.B. Vassallo,
dated February 3, 1984 (JPN-84-07).

Dear Sir:

This letter responds to a verbal request from the Nuclear Regulatory Commission (NRC) staff to provide information on the status of NUREG-0737 Item II.E.4.1 with regard to the James A. FitzPatrick facility. Although Item II.E.4.1 was previously discussed in References 1 and 2, the NRC requested that the Authority confirm the following items:

1. The JAF Containment Purge System meets General Design Criteria (GDC) 54 and 56;
2. The JAF procedures for use with degraded core have been reviewed;
3. JAF complies with Clarifications 1 and 2 of Item II.E.4.1 of NUREG-0737

The FitzPatrick Containment Purge System satisfies GDC-54 requirements for redundancy, leak detection, isolation, and test capability. Valve leakage has been found to be within acceptable limits.

GDC 56 states that each line that connects directly to the containment atmosphere and penetrates primary reactor containment shall be provided with two isolation valves, one inside and one outside containment unless it can be demonstrated that the containment isolation provisions are acceptable on some other defined basis.

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The FitzPatrick containment purge lines are provided with two isolation valves in series outside the primary containment. According to Standard Review Plan Section 6.2.4., this is acceptable to satisfy GDC 56.

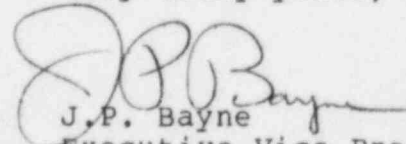
Emergency Operating/Special Procedure F-SP-02 "Post LOCA Venting of Containment and Operation of Main Steam Leakage Collection System" has been reviewed and updated to reflect the changes resulting from the modification, thereby complying with the requirements of NUREG-0737 Item II.E.4.1.

Clarifications 1 and 2 of Item II.E.4.1 of NUREG-0737 were addressed in References 1 and 2. Reference 1 provided the conceptual design for NRC pre-implementation review for Item II.E.4.1, and Reference 2 stated that the proposed modification was completed during the 1981-82 refueling outage except for providing Class 1E redundant power to two motor operated valves. This portion of the modification is scheduled to be completed prior to July 1, 1985.

Upon completion of this final portion of the modification, the Authority will consider NUREG 0737 Item II.E.4.1 to be complete for the FitzPatrick plant.

If you have any questions, please contact Mr. J. A. Gray, Jr. of my staff.

Very truly yours,



J.P. Bayne
Executive Vice President
Nuclear Generation

cc: Office of the Resident Inspector
U.S. Nuclear Regulatory Commission
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