

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) R.E. Ginna Nuclear Power Plant, Unit No. 1										DOCKET NUMBER (2) 0 5 0 0 0 2 4 4				PAGE (3) 1 OF 0 2				
TITLE (4) Inoperable Residual Heat Removal System																		
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)								
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)					
0	5	14	84	005	000	6	13	84					0 5 0 0 0					
OPERATING MODE (9) N		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																
POWER LEVEL (10) 0 1 0 0		20.402(b)				20.405(c)				50.73(a)(2)(iv)				73.71(b)				
		20.405(a)(1)(i)				50.36(a)(1)				50.73(a)(2)(v)				73.71(c)				
		20.405(a)(1)(ii)				50.36(a)(2)				X 50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)				
		20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)								
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)								
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)								
LICENSEE CONTACT FOR THIS LER (12)																		
NAME T.R. Schuler, Maintenance Manager										TELEPHONE NUMBER 3 1 1 5 5 1 2 4 1 - 4 4 4 1 6								
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																		
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS								
B	B P	- I S V	V 0 8 5	Y														
X	B P	- - 2 0 L	2 0 0	Y														
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR		
YES (If yes, complete EXPECTED SUBMISSION DATE)												X NO				0 0	0 0	0 0

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On May 14, 1984 while cooling down the Reactor Coolant System (RCS) to the cold shutdown condition for sludge lancing and crevice cleaning, MOV-700 (RCS Loop-A Residual Heat Removal Suction Valve) failed to stroke to the open position when actuated from the Control Room. Following manual unseating of the valve, maintenance personnel performed an inspection of the valve exterior. This inspection revealed that the packing gland flange had shifted out of the vertical position to a point where the flange was in contact with the valve stem. This could have caused a mechanical binding in the stem and torque-out of valve operator.

The valve was then stroked manually to verify no mechanical binding. The valve was then stroked twice electrically. The valve functioned satisfactorily with proper motor current readings, and acceptable opening and closing times, indicating no mechanical binding.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1) R.E. Ginna Nuclear Power Plant, Unit No. 1	DOCKET NUMBER (2) 0 5 0 0 0 2 4 4	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 4	— 0 0 5	— 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 365A's) (17)

A visual inspection of the valve stem and stem threads verified adequate cleanliness and lubrication. Torque switch settings were verified within the manufacturers design settings.

On 5/22/84 when the RCS was heating up to hot shutdown, the valve was again stroked to verify proper operation. Again, the valve functioned properly with proper motor current readings, and acceptable opening and closing times. Operation of this valve will continue to be monitored during the next cooldown of the RCS.



ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14649-0001

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ELECTRIC & STEAM PRODUCTION

TELEPHONE
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June 13, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: LER 84-005, Inoperable Residual Heat Removal (RHR)
System

R. E. Ginna Nuclear Power Plant, Unit No. 1
Docket No. 50-244

Document Control Desk:

In accordance with 10 CFR 50.73, Licensee Event Report System, item (a) (2) (vii), "any event where a single cause or condition caused at least one independent train or channel to become inoperable in multiple systems or two independent trains or channels to become inoperable in a single system designed to:

- (a) Shutdown the Reactor and maintain it in a safe shutdown condition;
- (b) Remove Residual Heat;
- (c) Mitigate the consequences of an accident."

The attached Licensee Event Report LER 84-005 is hereby submitted.

Very Truly Yours,

Roger W. Kober

Roger W. Kober

xc: Dr. Thomas E. Murley

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